

Neutron and X-ray computed tomography of a natural uranium tristructural isotropic (TRISO) fuel compact

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hanging the World's Energy Future

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INL/CON-25-84031-Revision-0

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Prepared for the U.S. Department of Energy Under DOE Idaho Operations Office Contract DE-AC07-05ID14517 March 28th, 2025

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- Idaho National Laboratory's mission is to *discover, demonstrate* and *secure* innovative *nuclear energy solutions*, other clean energy options and critical infrastructure
- Nondestructive examination techniques support this mission by:
 - Providing information for *modeling* fuel performance
 - Comparing pre- and post-irradiation effects
 - Informing destructive analysis
 - Saves time and money
- Quantify material performance in the harsh environment of a reactor to ensure safe commercial operations once deployed
- Ultimately, we aim to speed up the design, development, and deployment of nuclear energy technologies from conceptualization to commercialization



MFC Nuclear Research, Development & Demonstration Capabilities (*with other connected INL capabilities*)













Fabrication

- Experimental Fuel Facilities
- Fuels & Applied Science Building
- Fuel Manufacturing Facility
- Zero Power
 Physics Reactor
- Analytical
 Laboratory
- Advanced Fuels Facility

Fresh Fuel Characterization

- Fuels & Applied Science Building
- Analytical Laboratory
- Experimental Fuel Facilities

Irradiation

- Transient Reactor Experiment & Testing (TREAT)
- Neutron Radiography Reactor (NRAD)
- Advanced Test Reactor (ATR)
- Offsite Reactors

Post-Irradiation Examination & Characterization

- •Hot Fuel Examination Facility
- Irradiated Materials Characterization Lab
 Fuel Conditioning
- Facility

 Analytical Laboratory
 Fuels & Applied
- •Electron Microscopy
- •Neutron Radiography Reactor (NRAD)

Space Nuclear Power and Isotope Technologies

- Space & Security Power Systems Facility
- Engineering Development Lab
- Idaho Nuclear Technology & Engineering Center

Advanced Reactor Demonstration Test Beds

- TREAT micro-Reactor Experiment Cell
- Laboratory for Operations and Testing in the US
- Demonstration of Microreactor Experiments

TRISO Fuel and Compacts

- TRISO- TRi-structural ISOtropic particle fuel
- PIE of individual particles investigates changes in individual particles layers
 - Thickness changes, buffer delamination, etc.
 - Kernel porosity and fission product migration
- PIE XCT of intact fuel compacts and loose particles yields different information. Fuel compact examination provides:
 - Kernel morphology
 - Variations morphology as a function of spatial position
 - Quantification of the buffer fracture frequency
 - Preservers compact for additional exams
 - Impossible following traditional examinations





AGR Program TRISO Particle ~855 µm Diameter





- Best spatial resolution to date: 270 nm/voxel
- Hottest sample to date: 120 R/hr on contact
- Instrument is maintained as radiologically clean





2D slices of a 3D reconstructed volume showing a) a lowenergy (40 keV) scan on an irradiated AGR-2 TRISO particle, b) the corresponding high-energy (110 keV) scan of the same particle, c) the images fused together, and d) a 3D rendering of the particle using information from both datasets.



William Chuirazzi, Joshua J. Kane, Nikolaus L. Cordes, John D. Stempien, Rahul R. Kancharla, and Fei Xu. "Seeing the whole picture: Methods for getting the most from micro X-ray computed tomography of TRISO nuclear fuel particles." *Tomography of Materials and Structures* 2 (2023): 100005.

TRISO Compact Examination



XCT Renderings



Quantitative Results

- Quantify kernel information
 - Equivalent diameter
 - Sphericity
 - Kernel position
 - Packing fraction
- Useful to compare pre- and postirradiation to examine changes
 - Kernel swelling
 - Aspherical particles



0

0.99

0.991 0.992

0.993

0.994

0.995 0.996

Kernel Sphericity

0.997

0.998

0.999

1

More

Need for Neutron CT

- Can we see defects in the matrix of the compacts nondestructively?
- Informs structural integrity of the compact before and after irradiation
- If features of interest (i.e. cracks) are found, their location can be used to determine potential causes
- X-ray attenuation introduces too much noise for matrix examination
 - Advanced reconstruction and artifact reduction techniques cannot sufficiently resolve features in the matrix
- Neutrons may offer a solution
- Used an unirradiated, natural Uranium compact as a proof-of-principal





MARS Beamline

- Multimodal Advanced Radiography Station at HFIR's CG-1D Beamline
- Polychromatic beam of cold neutrons (peak wavelength 2.6 Å)





MARS Imaging Conditions

- Absolute neutron flux was ~5x10⁶ n/cm²/s
- Pixel size is ~16 um/pixel
 - Spatial resolution is ~48 um
- L/D ~600
- sCMOS camera
- 1091 Radiographs captured
- SIRT Reconstruction algorithm
- Ring artifact removal but otherwise no image processing
 - bm3d ring removal



nCT Results

- Difficult to get information about individual particles
- Crack within matrix can clearly be seen across multiple slices
- Can provide useful method for matrix inspection











nCT Renderings

1

1. 1





Crack Identification







Future Capabilities – nCT for Multiple FOV's

- Different applications require different FOV's
- A multi-FOV imaging system would cover all applications possible in the NRS beam
- Targeting $\sim 20 \ \mu m$ effective spatial resolution (7 μm pixel)









Conclusions

- Neutron CT successfully identified cracking in the graphite matrix
- X-ray and neutron CT perform complementary examinations of fuel compacts
 - XCT: Kernel information
 - **nCT**: Matrix information

Potential Future Work:

- nCT of irradiated compact
- XCT/nCT data registration
- Isotopic fission product mapping (nGI/ToF)



Calculations performed with ORNL's NEUIT tool: https://neuit.ornl.gov/transmission

Acknowledgements

- Thanks to Phil Winston, Dave Laug, BJ Camphouse, and John Stempien for programmatic support and resources in providing samples and coordinating sample transfer between INL and ORNL.
- Thanks to the HFIR support staff for facilitating the experiment.
- This work was supported by Battelle Energy Alliance, LLC, under Contract No. DE-AC07–05ID14517 with the U.S. Department of Energy.
- This research used resources at the High Flux Isotope Reactor a DOE Office of Science User Facility operated by the Oak Ridge National Laboratory. The beam time was allocated to MARS (CG-1D) on proposal number IPTS-34736.

Idaho National Laboratory

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