



Out-of-pile and postirradiated examination of lanthanide and lanthanide-palladium interactions for metallic fuel

February 2021

Changing the World's Energy Future

Michael T Benson, Jason Harp, Kevin R Tolman, Yi Xie, Tiankai Yao, Karen E Wright, James A King, Ayman Hawari, Qingsheng Cai



DISCLAIMER

This information was prepared as an account of work sponsored by an agency of the U.S. Government. Neither the U.S. Government nor any agency thereof, nor any of their employees, makes any warranty, expressed or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness, of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. References herein to any specific commercial product, process, or service by trade name, trade mark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the U.S. Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the U.S. Government or any agency thereof.

Out-of-pile and postirradiated examination of lanthanide and lanthanide-palladium interactions for metallic fuel

**Michael T Benson, Jason Harp, Kevin R Tolman, Yi Xie, Tiankai Yao, Karen E
Wright, James A King, Ayman Hawari, Qingsheng Cai**

February 2021

**Idaho National Laboratory
Idaho Falls, Idaho 83415**

<http://www.inl.gov>

**Prepared for the
U.S. Department of Energy
Under DOE Idaho Operations Office
Contract DE-AC07-05ID14517**

1 **Out-of-pile and postirradiated examination of lanthanide and lanthanide-palladium interactions for**
2 **metallic fuel**

3
4 Michael T. Benson^a, Jason M. Harp^b, Yi Xie^c, Tiankai Yao^a, Kevin R. Tolman^a, Karen E. Wright^a, James
5 A. King^a, Ayman I. Hawari^d, Qingsheng Cai^d

6
7 ^aIdaho National Laboratory, P.O. Box 1625, MS 6188, Idaho Falls, ID 83415

8 ^bOak Ridge National Laboratory, Oak Ridge, TN 37831

9 ^cPurdue University, 516 Northwestern Ave., West Lafayette, IN, 47906

10 ^dNorth Carolina State University, P.O. Box 7909, Raleigh, NC 27695

11
12 **Corresponding Author Information**

13 Michael T. Benson, phone: (208) 533-8870, email: michael.benson@inl.gov

14
15 **Abstract**

16 Palladium is being investigated as a fuel additive to bind with and potentially immobilize
17 lanthanide fission products. A primary cause of fuel-cladding chemical interaction (FCCI) is the
18 lanthanide fission products migrating to the fuel periphery and interacting with the cladding. This
19 interaction will lead to wastage of the cladding and eventually to a cladding breach. Palladium has
20 previously been identified as a promising additive used to prevent or decrease FCCI by reacting with the
21 lanthanide fission products. In the current study, an alloy cast from the four highest abundant lanthanides
22 found in irradiated metallic fuel, Nd, Ce, Pr, and La, with and without Pd, has been characterized using
23 neutron diffraction, scanning electron microscopy, and electron probe microanalysis. In the lanthanide-Pd
24 intermetallic compounds, all of the constituent compounds, i.e. Nd-Pd, Ce-Pd, La-Pd and Pr-Pd are
25 known. There is very good agreement, both structurally and compositionally, between the out-of-pile
26 lanthanide alloy and lanthanide fission products characterized in irradiated fuels. In both cases, the
27 lanthanide elements form a solid solution in a hexagonal crystal structure. The out-of-pile lanthanide alloy
28 follows Vegard's Law, with the measured and calculated (weighted average of constituents) lattice
29 parameters being within 1% for both the a and c parameters. Pd bonds with the lanthanides (Ln) forming
30 the phases LnPd and Ln₇Pd₃. The results indicate the properties of lanthanide compounds in irradiated
31 metallic fuel can be reliably simulated in out-of-pile experiments.

32
33 **Keywords:** metallic nuclear fuel, fuel-cladding chemical interaction, neutron diffraction, lanthanide
34 compound

35

1. Introduction

Fuel-cladding chemical interaction (FCCI) occurs when the nuclear fuel or fission products react with the cladding material. This interaction limits fuel performance by restricting the maximum temperature at which fuel can be operated. As more lanthanide fission products are created at higher burnups this performance limitation becomes more severe. Due to this, there are numerous investigations, both in-pile and out-of-pile, in the literature, as well as three recent review articles. Keiser recently published a historical perspective on FCCI, discussing the efforts over the last 70 years to understand and prevent FCCI to allow higher burnup and higher reactor operating temperatures [1], Matthews *et al.* published a review of FCCI in U-Pu-Zr metallic fuel [2], and Xie et al. published a review of the lanthanide migration mechanism in U-Zr metallic fuels [3]. A major cause of FCCI in U-Zr or U-Pu-Zr fuels during irradiation is fission product lanthanides (Ln), which move down the temperature gradient to the fuel periphery [2]. The result of this interaction is embrittlement and degradation of the cladding, which will eventually lead to rupture of the fuel rod.

Lanthanides can be present in a fuel during irradiation as fission products, but can also be present if the fuel was fabricated with recycled U. A small amount of lanthanides are expected in metallic fuel fabricated with recycled uranium [4]. Controlling FCCI in this system is even more important due to the early presence of lanthanides in the fuel. As soon as the fuel contacts the cladding due to swelling, at roughly 1-2% burnup [5], there are lanthanides available to react with the cladding and cause FCCI. This is opposed to the much slower generation of lanthanides solely from the fission process in a fuel fabricated without pre-existing lanthanides.

Several methods are being investigated to decrease or prevent FCCI, such as barrier foils [6], coatings [7][8], and additive materials [9-12]. After considering ways to bind lanthanides as stable intermetallics, a set of criteria were developed that identified Pd as a promising additive, especially since it is already a fission product [13]. The amount of fission product Pd is insufficient to bind the lanthanides, though, requiring additional Pd to be added to the alloy. In the chemical analysis of a U-10Zr fuel pin irradiated in EBR-II [13], the amount of Pd measured (0.05g) is very small compared to the quantity of lanthanides (1.73g, in an 81.47g sample). Recent work using Pd as an additive has shown promising results, both in-pile [14] and out-of-pile [13][15-17]. Lanthanide-Pd compounds have also been observed in irradiated U-10Zr when there is no added Pd, but only present as a fission product [18], lending credibility to this mitigation strategy. These compounds have also been observed in other metallic fuel alloys after irradiation [19-24]. The efficacy of fuel additives, specifically Pd, has been discussed in previous investigations [3][15][13].

The importance of FCCI in limiting the lifespan of nuclear fuel due to fission product lanthanides is known [2], although there has been no characterization of the lanthanides involved. No prior work in the literature has been performed to characterize the chemical structure of mixed lanthanide phases or mixed

71 lanthanide-palladium phases. This work is the characterization of two alloys, with one being a mix of 4
 72 lanthanides, Nd, Ce, La, and Pr, and the other alloy being those lanthanides mixed with Pd. These alloys
 73 were characterized using neutron diffraction, scanning electron microscopy (SEM), and electron probe
 74 micro-analyzer (EPMA). These out-of-pile characterizations are compared against the characterization of
 75 lanthanides and lanthanide-Pd compounds in two irradiated fuels. The first fuel is an annular, helium
 76 bonded U-10Zr fuel pin irradiated in the Advanced Test Reactor to 3.3 % burnup [14], and the second is a
 77 solid, sodium bonded U-10Zr fuel pin irradiated to 5.7 % burnup (localized burnup) in the Fast Flux Test
 78 facility [16]. The composition and ratio of lanthanides used in the out-of-pile characterizations are based
 79 on elemental analysis of an irradiated U-10Zr EBR-II fuel pin [13], with the four most prevalent
 80 lanthanides included in the mix. In the chemical analysis, only elements above 0.01g were included,
 81 which encompassed 6 lanthanides (La, Ce, Pr, Nd, Pm, Sm). The four highest abundance (Nd, Ce, La, and
 82 Pr) lanthanides are 92% of the measured lanthanide fission products. The ratio obtained from the EBR-II
 83 fuel is approximately 53Nd-25Ce-16Pr-6La, in wt. %.

84

85 2. Experimental Methods

86 Post-irradiation examinations (PIE) were performed on two U-10Zr fuel samples, focusing on phase
 87 analysis and the chemical composition of Ln precipitates. A rough estimate, based on Origen2
 88 calculations [25], of the expected quantity of Pd, Nd, Ce, Pr, and La produced as fission products in each
 89 irradiated fuel is listed in Table 1, along with basic details of the fuel. The total lanthanides produced are
 90 1.2E-03 mol/mol fuel in the annular U-10Zr fuel pin, and 3.2E-02 mol/mol fuel in the MFF-3 U-10Zr fuel
 91 pin.

92

93 Table 1. Irradiated fuel details, with atomic concentration of selected fission products in the fuel (in
 94 mol/mol fuel).

	Burnup	Bonding agent	Pd	Nd	Ce	Pr	La	Cs	Ba
Annular U-10Zr	3.3 at. %	Helium	7.3 E-05	4.9 E-04	3.8 E-04	1.6 E-04	1.7 E-04	5.1E-04	1.9E-04
MFF-3 U-10Zr	5.7 at. %	Sodium	2.8 E-03	1.4 E-02	8.4 E-03	4.2 E-03	4.5 E-03	1.2E-02	6.6E-03

95

96 The first irradiated sample is an annular U-10Zr fuel that was irradiated to 3.3 at. % burnup in the
 97 Advanced Test Reactor (ATR). Initial characterization of this fuel pin (not including advanced techniques
 98 such as electron microscopy) has been previously reported [14]. As described in the previous report, this
 99 fuel pin was removed from the reactor early due to variations in the inner diameter of the cladding
 100 causing the fuel to run hotter than expected. Helium has very poor thermal conductivity, so a helium-
 101 bonded fuel cannot tolerate large fuel-cladding gaps. The issues in the rodlet assembly were corrected in

102 subsequent annular fuel rods, and this fuel pin, even though it ran hot, has provided an initial look at this
 103 novel fuel design. A transmission electron microscope (TEM) sample that covers both the U matrix and
 104 Ln precipitate was prepared using focused ion beam (FIB) in a FEI Quanta 3D FEG Dual Beam
 105 (SEM/FIB). A 2 KeV Ga ion beam was used for final polishing prior to transferring the sample into the
 106 TEM. Electron diffraction (ED) patterns and STEM EDS data were collected at room temperature using a
 107 Thermo Fisher™ Talos F200X scanning/TEM (STEM) operated at 200 KeV. EDS data was collected in
 108 the scanning mode using four Super-X windowless EDS detectors based on ChemiSTEM technology,
 109 with the beam size < 1 nm. SEM images of the irradiated annular U-10Zr fuel were also obtained on the
 110 FEI Quanta 3D FEG Dual Beam (SEM/FIB).

111 The second post-irradiation sample is U-10Zr, from the MFF-3 assembly irradiated in the Fast Flux
 112 Test Facility (FFTF) [26]. The post-irradiation examination (PIE) of this specific fuel pin has been
 113 previously reported [18][27]. This assembly had a peak inner cladding temperature of 643 °C and
 114 operated to 13.8 at. % burnup. The slice used for SEM analysis was taken at x/L=0.98 (fuel pin
 115 length=91.4cm) from fuel pin serial number 193045. The local burnup for the sample was 5.7 at. % and
 116 the time averaged temperature of the inner cladding for the sample was 615 °C [27].

117 Two alloys were cast, one with four lanthanides, Nd, Ce, Pr, and La, and another alloy containing
 118 those lanthanides and Pd. The alloy compositions are listed in Table 2. Materials were obtained from Alfa
 119 Aesar and used as received. The lanthanides were obtained as rods, packaged in mylar under argon. Even
 120 packaged under argon, some oxidation was visible on the surface of the Ce and La rods. There were no
 121 attempts to clean the rods, and they were used as received. The specific lanthanide composition is roughly
 122 based on the chemical analysis of an EBR-II U-10Zr irradiated fuel pin (8% burnup) [13], using the ratio
 123 of the four most prevalent lanthanides. The Pd content is based on the SEM analysis of a Pd-Ln
 124 precipitate in a U-10Zr fuel pin irradiated in FFTF, as described more fully below. For simplicity, the
 125 alloys will be referred to as the Ln alloy, or the Ln-Pd alloy.

126

127 Table 2. Alloy compositions

Alloy	Composition (wt. %)	Composition (at. %)
Ln	53Nd-25Ce-16Pr-6La	52.3Nd-25.4Ce-16.2Pr-6.1La
Ln-Pd	39.8Nd-18.8Ce-12Pr-4.0La-25.0Pd	36.2Nd-17.6Ce-11.2Pr-4.3La-30.7Pd

128

129 All casting operations were carried out in an arc-melter within an argon atmosphere glovebox with
 130 high purity argon as a cover gas. To cast the Ln alloy, the appropriate amount of Nd, Ce, Pr, and La were
 131 arc melted together in one step. To cast the Ln-Pd alloy, the appropriate amount of a Ln pre-alloy was
 132 mixed with Pd. In each case, after melting the constituents together, the resulting button was flipped and
 133 re-melted three times to ensure homogeneity, which was verified visually when performing the SEM
 134 analysis. The buttons were cast into 5 mm diameter pins.

135 Approximately 1.5” were taken from each pin and loaded into separate 0.250” x 2.00” vanadium
136 sample cans (Metal Technologies) for neutron diffraction studies. All processing of the lanthanides, from
137 opening the mylar packages to loading the samples cans, was performed in an argon glovebox. Scanning
138 electron microscopy (SEM) was performed on a section of the Ln-Pd pin mounted in a 31.8 mm diameter
139 phenolic metallographic (met) mount filled with epoxy. Samples were polished by grinding the surfaces
140 flat with SiC grinding paper followed by polishing with polycrystalline diamond suspensions, starting
141 with 9 μm, then 3 μm, and finally 1 μm. The polished sample was analyzed with a sputtered coating of
142 approximately 15 nm carbon to control charging of the met mount.

143 Two instruments were utilized for SEM analysis. The SEM instrument used for non-irradiated
144 analysis was a JSM-7600f SEM, while the MFF-3 sample irradiated at FFTF was analyzed using a
145 JSM-7000f SEM. Both instruments were manufactured by the Japan Electron Optics Laboratory (JEOL),
146 and are hot field emission SEMs equipped with Oxford Instruments X-Max 20 silicon drift energy
147 dispersive X-ray spectrometers (EDS) and Nordlys II F+ electron back scattered diffraction cameras.
148 Both SEMs were operated at an accelerating voltage of 20 keV. The X-ray spectrometers are controlled
149 by Oxford INCA software (v. 4.15, part of the Oxford Microanalysis Suite Issue 18d + SP 4), which also
150 provides image acquisition capabilities. Spectra were collected over the energy range 0 – 20 keV, which
151 covers characteristic X-ray energies from all analytes. Spectra were quantified using so-called
152 “standardless” analysis, which uses a stored library of reference spectra to quantify unknown spectra
153 rather than physical standards. This method is generally accurate to the 0.1 to 0.5 w/w% range, depending
154 on sample and microscope (observation) conditions. Quantitative results for the Ln-Pd alloy were
155 normalized to Pd, Nd, Ce, Pr, and La.

156 Compositional analyses were acquired on a CAMECA SX100R electron probe microanalyzer
157 (EPMA) equipped with four tunable wavelength dispersive spectrometers. A focused beam with an
158 accelerating voltage of 20 kV and current of 101 nA was used on each point. Analytical conditions are
159 listed in Table 3. Quantitative point analysis was performed using Probe for EPMA v. 11.8.8. The mass
160 absorption coefficients dataset was FFAST [28], and the phi-rho-Z algorithm utilized was Pouchou and
161 Pichoir-Full [29]. Results are from single points and detection limits ranged from 0.035 wt. % for O α to
162 0.043 wt. % for La α to 0.070 wt. % for Ce α . The single point detection limit calculation (Equation 1)
163 is based on the standard counts and the unknown sample background counts and include the magnitude of
164 the ZAF correction factor. The calculation is adapted from Ref. [30], and is useful in that it can be used
165 even on inhomogeneous samples and can be quoted as the detection limit in weight percent for a single
166 analysis line with a confidence of 99% (assumes 3 standard deviations above the background).

167
168
$$CDL = (ZAF) (3/I_s) (SQRT (I_b * 100)) \quad (Eq. 1)$$

169

170 CDL is the calculated detection limit with a 99% (3-sigma) confidence, ZAF is the ZAF correction
 171 factor for the sample matrix, I_s is the count rate (cps/nA) on the pure element standard (apply std k-
 172 factor), and I_b is the background count rate (cps/nA) on the unknown sample. In Equation 1, before
 173 calculating the SQRT of the background count rate, the count rate (cps/nA) must first be denormalized to
 174 raw counts with the beam current and integration time for the unknown intensity. After calculating the
 175 SQRT, the background must be re-normalized to cps/nA before calculating the above k-ratio I_B/I_S .

176 After obtaining quantitative X-ray maps on the EPMA, an analysis of each of the 47,000 map pixels
 177 was performed using a Hartigan-Wong k-means clustering algorithm, providing a modal analysis of
 178 phases and quantities present. In this algorithm, the user specifies the components to use for classification,
 179 the number of clusters and the desired precision. The program then calculates a statistical representation
 180 of the area percentage of each cluster, which corresponds to a phase. After inputting the density of each
 181 phase, the program calculates the mass percent of each phase in the X-ray map.

182

183 Table 3. Analytical parameters employed for EPMA quantitative pint analysis.

X-ray	Crystal	Peak Time (s)	Background time (s)	Corrected overlap	Standard
O $K\alpha$	PC1	30	20		NdP ₅ O ₁₄
Pd $L\alpha$	PET	20	20		Pd
Nd $L\alpha$	LiF	20	20		NdP ₅ O ₁₄
Ce $L\alpha$	LiF	30	20		CeP ₅ O ₁₄
Pr $L\alpha$	LiF	30	20	La	PrP ₅ O ₁₄
La $L\alpha$	LiF	60	60	Nd	LaP ₅ O ₁₄

184

185 Neutron diffraction measurements were carried out at the PULSTAR Neutron Powder
 186 Diffractometer at North Carolina State University. The PULSTAR reactor is a swimming pool reactor
 187 with maximum power 1 MW. The diffractometer is located at beam tube 4 of the reactor. A double
 188 focusing monochromator, which can focus two-dimensional neutrons to the sample position, is employed
 189 in this facility. The flux of neutrons with wavelength 1.489 Å is 0.5×10^5 n/cm²s at the sample position. A
 190 15 position sensitive detector array attached to a moveable cassette can span 20° (a section) and can be
 191 moved from 15° to 115° to cover the entire scattering angle of 5°-125°. A fission chamber located at the
 192 beam before the sample is employed as the monitor. The detectors collect data at each section for a fixed
 193 monitor count to ensure the incoming neutrons at the sample position is the same for each section. The
 194 data from the 6 sections are collated into the full scattering angle range data. The neutron powder
 195 diffraction data was analyzed using Rietveld method [31] with the GSAS program [32].

196

197 **3. Results**

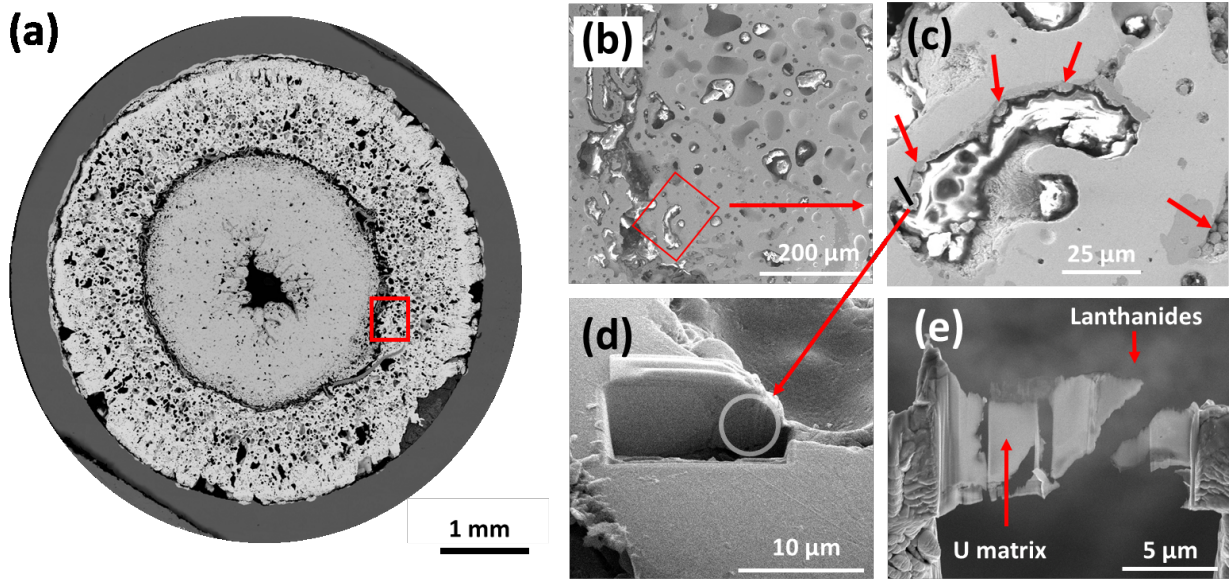
198 **3.1 PIE of irradiated annular U-10Zr**

199 Basic PIE has been reported for the annular U-10Zr fuel pin, including neutron radiography,
200 evaluation of the distribution of gamma-ray emitting radionuclides, dimensional inspection, fission gas
201 release analysis, optical microscopy, and chemical analysis [14]. Advanced PIE (SEM and TEM) of the
202 fuel pin [33] and the fuel-cladding interface [34] have also been reported. This manuscript focuses on
203 characterization of lanthanide, and lanthanide-Pd compounds, thus only the characterized lanthanide
204 precipitates are within the scope of this investigation.

205 A montage image of the annular U-10Zr fuel pin section is shown in Figure 1a, along with the
206 location where the lamella was lifted, indicated by the red rectangle. The location is just outside of the
207 central, Zr-rich region. Originally this was the annulus of the fuel, but filled in during irradiation as the
208 fuel swelled and fission gas bubbles developed. NOTE: The sample was polished prior to loading into the
209 FIB to remove the oxide layer. Due to this, the montage image shown in Figure 1a does not exactly match
210 the image shown in Figure 1b. The location of the lamella is accurate based on remaining larger features
211 on the surface.

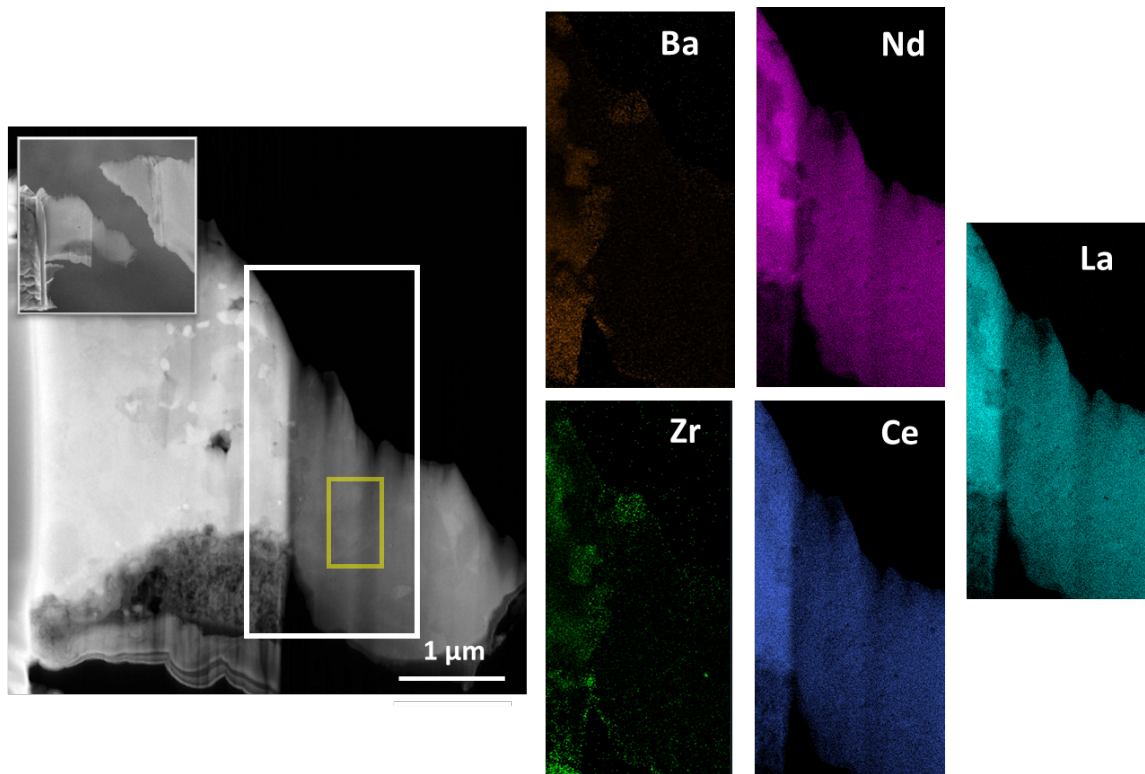
212 Figure 1b and Figure 1c show magnified images of where the lamella was lifted. The lanthanides
213 were observed as nodules around the periphery of pores, as indicated in Figure 1c with red arrows. The
214 lamella was lifted from the edge of a pore, which included a lanthanide nodule and U matrix. The thinned
215 lamella is shown in Figure 1e, with the lanthanide nodule on the right side of the lamella, while the left
216 side is U matrix. The lower right side of the lamella split away during thinning, and it is this small section
217 of the lanthanide nodule that has been further investigated. EDS mapping of this section is shown in
218 Figure 2 for Ba, Nd, Ce, Zr, and La, with the region indicated by the white rectangle. Ba is found on the
219 left side of this region, between the lanthanide nodule and the U matrix. The lanthanides are evenly
220 distributed throughout the nodule, with the only variations occurring where there is a higher concentration
221 of Ba. The yellow rectangle indicates the location of quantitative STEM EDS analysis, with the results
222 listed in Table 4.

223



224

225 Figure 1. a. Montage image of irradiated annular U-10Zr. Red rectangle indicates location of image
 226 shown in 1b. Red rectangle in 1b indicates location of image shown in 1c. The black line indicates
 227 the location of the lamella. The red arrows in 1c indicate nodular lanthanide precipitates. 1e shows an image
 228 of the final lamella after thinning.
 229

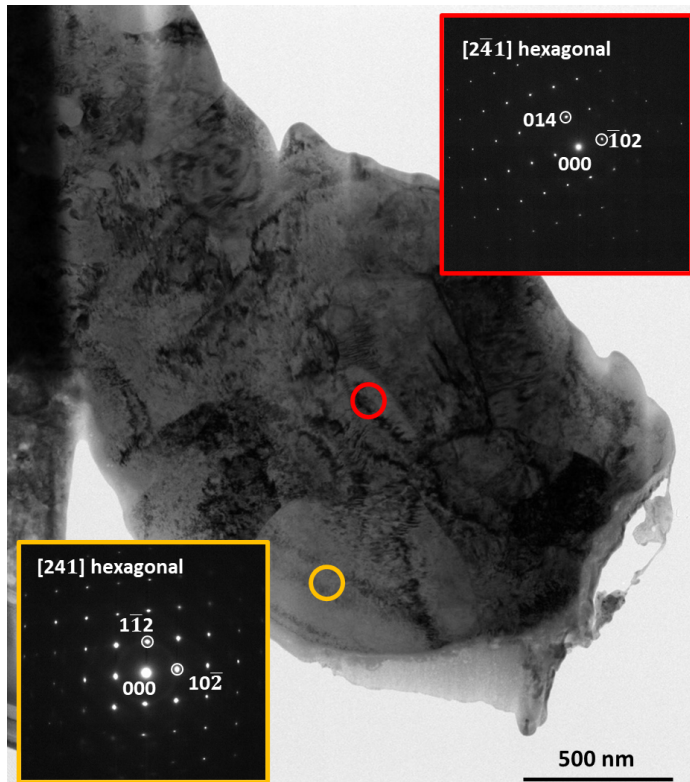


230 Figure 2. TEM EDS mapping of lanthanide precipitate. The white rectangle indicates the mapping region,
 231 and the yellow rectangle indicates the location of quantitative EDS analysis, with the results listed in
 232 Table 4.
 233
 234

235 Table 4. STEM EDS analysis for region indicated by yellow rectangle in Figure 2.

element	at. %
Y	2.1
Zr	7.7
La	14.2
Ce	21.5
Pr	4.1
Nd	48.8
Sm	1.5
U	0.1

236
 237 To identify phase, diffraction patterns were taken from the Ln grains. A brightfield TEM image
 238 showing the location of two collected diffraction patterns is shown in Figure 3, with the diffraction
 239 patterns included as inserts. Indexing of these patterns indicates the Ln precipitates are in a hexagonal
 240 crystal structure, with lattice parameters calculated to be $a = 3.89 \text{ \AA}$ and $c = 12.55 \text{ \AA}$, based on the
 241 interplanar distance in the diffraction patterns. The diffraction patterns shown are very clean, with no
 242 indication of oxidation during sample transfer between the FIB and TEM. Oxidation is always a concern
 243 due to the extremely oxophilic nature of the lanthanides.
 244

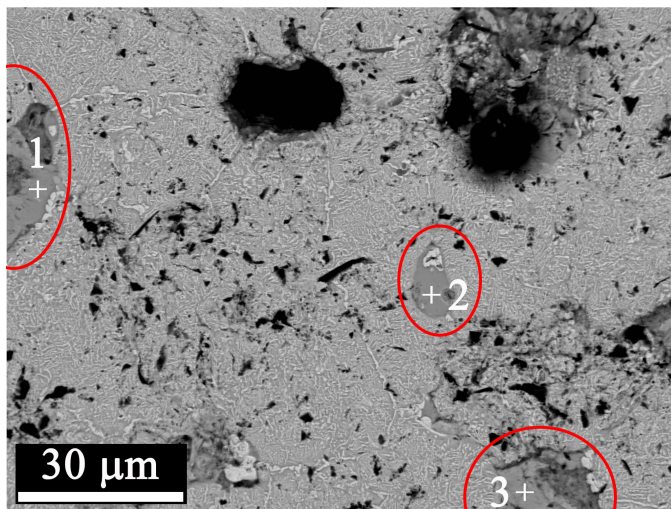


245 Figure 3. TEM brightfield image showing locations (circles) of diffraction analysis in the lanthanide
 246 precipitate, with the diffraction patterns included.
 247
 248

249 **3.2 PIE of irradiated solid U-10Zr**

250 The full characterization of U-10Zr MFF3 pin #193045 has been previously reported [18][27], and
251 is only included here as an example of Pd-lanthanide precipitates in an irradiated fuel. The post-
252 irradiation microstructure for this fuel pin is shown in Figure 4. This SEM image was obtained from the
253 central, Zr-rich region of the fuel pin. The majority of the microstructure is a mixture of α -U and δ -UZr₂,
254 with this region of the fuel being approximately 30 to 35 wt. % Zr by EDS analysis. The enhanced
255 concentration of Zr in this area is due to the well-known phenomenon of Zr redistribution in irradiated U-
256 Zr fuels [18]. A key feature of the microstructure shown in Figure 4 is the dark grey precipitates
257 highlighted by the red circles. The composition of the precipitates, as determined by EDS analysis, is
258 listed in Table 5. The presence of these lanthanide-Pd precipitates in this sample with fission product
259 generated Pd is encouraging for the feasibility of this FCCI mitigation strategy. Due to the low fission
260 yield of Pd compared to the lanthanides, not all of the observed lanthanide deposits in this fuel contained
261 Pd.

262



263

264 Figure 4. SEM backscatter image of FFTF U-10Zr fuel, post-irradiation [18]. The red circles indicate Pd-
265 Ln precipitates. Corresponding EDS data are listed in Table 5.

266

267 Table 5. EDS data for points shown in Figure 4. Values in at. %.

	U	Pd	Nd	Ce	La	Y	Sm
1	0	31	35	13	9	8	4
2	1	32	36	14	8	9	0
3	0	31	38	15	10	6	0

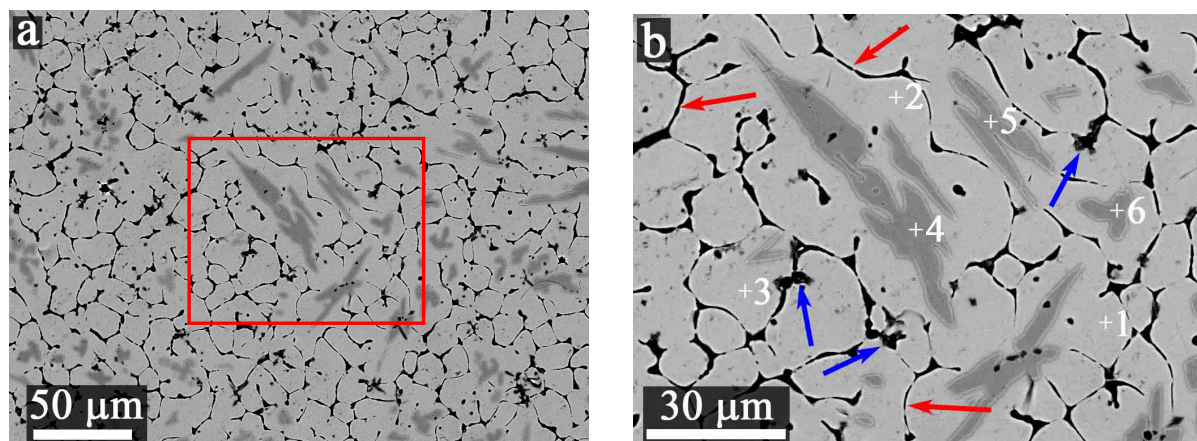
268

269 **3.3 SEM characterization of non-irradiated samples**

270 A representative SEM backscatter image of the Ln-Pd alloy is shown in Figure 5a, with a
271 magnified image shown in Figure 5b. The EDS points shown, with data listed in Table 6, are for the

272 matrix region and the precipitates. The microstructure shows large Pd-rich inclusions, and black
 273 intergranular regions. The intergranular boundaries contain lanthanide oxides. Although the lanthanide
 274 rods were packaged under argon in mylar bags, Ce and La still had some oxide on the surface,
 275 significantly more than Nd and Pr. No attempts to clean the raw materials were made and are the likely
 276 source of the oxide. The oxides are mixed throughout the pins, indicating the oxide was present prior to or
 277 during arc melting. The composition of the oxides varied, although in general the intergranular oxides,
 278 indicated with red arrows in Figure 5b, are comprised of Ce, Pr, and La. The larger black spots, indicated
 279 with blue arrows, are Nd_2O_3 .

280



281 Figure 5. a. and b. SEM backscatter images for the Ln-Pd alloy. The red rectangle in 5a indicates the
 282 magnified region shown in 5b. Corresponding EDS results are listed in Table 6. Red arrows indicate
 283 representative regions of Ce, La, and Pr oxides. Blue arrows indicate representative areas of Nd_2O_3 .
 284
 285

286 Table 6. EDS analysis results for points shown in Figure 5b. Values shown in at. %.

	Nd	Ce	Pr	La	Pd	Phase ^a
1	35	18	10	5	32	Ln_7Pd_3
2	35	18	11	4	32	Ln_7Pd_3
3	35	18	10	5	32	Ln_7Pd_3
4	30	9	7	2	52	LnPd
5	30	9	7	2	52	LnPd
6	30	9	7	2	52	LnPd

287 ^a Suggested phase based on EDS analysis

288

289 3.4 EPMA characterization of non-irradiated samples

290 In the SEM EDS analysis, differences in $\text{L}\alpha$ peak energy between each of the lanthanides (La to
 291 Ce, Ce to Pr, and Pr to Nd) is consistent at roughly 0.19 keV, causing a significant amount of peak
 292 overlap. There is also significant peak overlap for the lanthanides in the neutron diffraction data discussed
 293 below. In order to accurately determine the elemental composition of the phases, quantitative EPMA was
 294 performed using standards for each element.

295 EPMA analysis was performed on the two primary phases in the Ln-Pd alloy, shown in Figure 5b,
 296 i.e. the light grey areas (Ln_7Pd_3), and the dark grey areas (LnPd), and on the intergranular oxide. For the
 297 two primary phases, ten spots were quantitatively analyzed, while for the oxide, three spots were
 298 analyzed, with the results listed in Table 7. The oxide analysis listed in Table 7 does not include oxygen,
 299 due to interference from surface oxidation. For each of the phases, the ratio of lanthanides present in the
 300 phase is used in the discussion. For this reason, excluding oxygen, which will be erroneous due to surface
 301 oxidation, does not affect the resulting discussion. Using a cluster analysis, as described in the
 302 experimental section, the amount of each phase on the surface was determined to be 79.0 % of the Ln_7Pd_3
 303 phase, 12.0 % of the LnPd phase, and 9.0 % of the oxide phase.

304

305 Table 7. EPMA measured elemental compositions in the Ln-Pd alloy. All values in at. %.

Point	Nd	Ce	Pr	La	Pd
Ln_7Pd_3					
1	36.9	17.2	11.4	4.4	30.1
2	36.8	17.3	11.5	4.4	30.0
3	37.1	17.2	11.4	4.4	29.9
4	37.1	17.3	11.3	4.4	29.9
5	37.2	17.2	11.4	4.5	29.8
6	36.8	17.6	11.3	4.5	29.8
7	37.3	17.3	11.4	4.4	29.6
8	37.3	17.2	11.4	4.4	29.8
9	37.0	17.2	11.5	4.4	29.9
10	37.1	17.3	11.4	4.3	30.0
Average	37.1	17.3	11.4	4.4	29.9
LnPd					
1	31.4	9.0	7.8	2.2	49.6
2	31.3	9.2	7.7	2.2	49.7
3	31.2	9.1	7.7	2.2	49.7
4	31.2	9.0	7.7	2.3	49.8
5	31.3	8.9	7.7	2.2	49.9
6	31.4	9.0	7.7	2.2	49.7
7	31.5	8.8	7.7	2.1	49.8
8	31.9	9.5	7.8	2.3	48.6
9	31.5	8.8	7.8	2.1	49.8
10	31.6	9.2	7.7	2.2	49.3
Average	31.4	9.1	7.7	2.2	49.6
Oxide^a					
1	37.3	36.4	16.3	7.9	2.1
2	38.3	35.6	16.3	7.8	2.0
3	37.6	35.4	16.3	7.8	2.9
Average	37.7	35.8	16.3	7.9	2.3

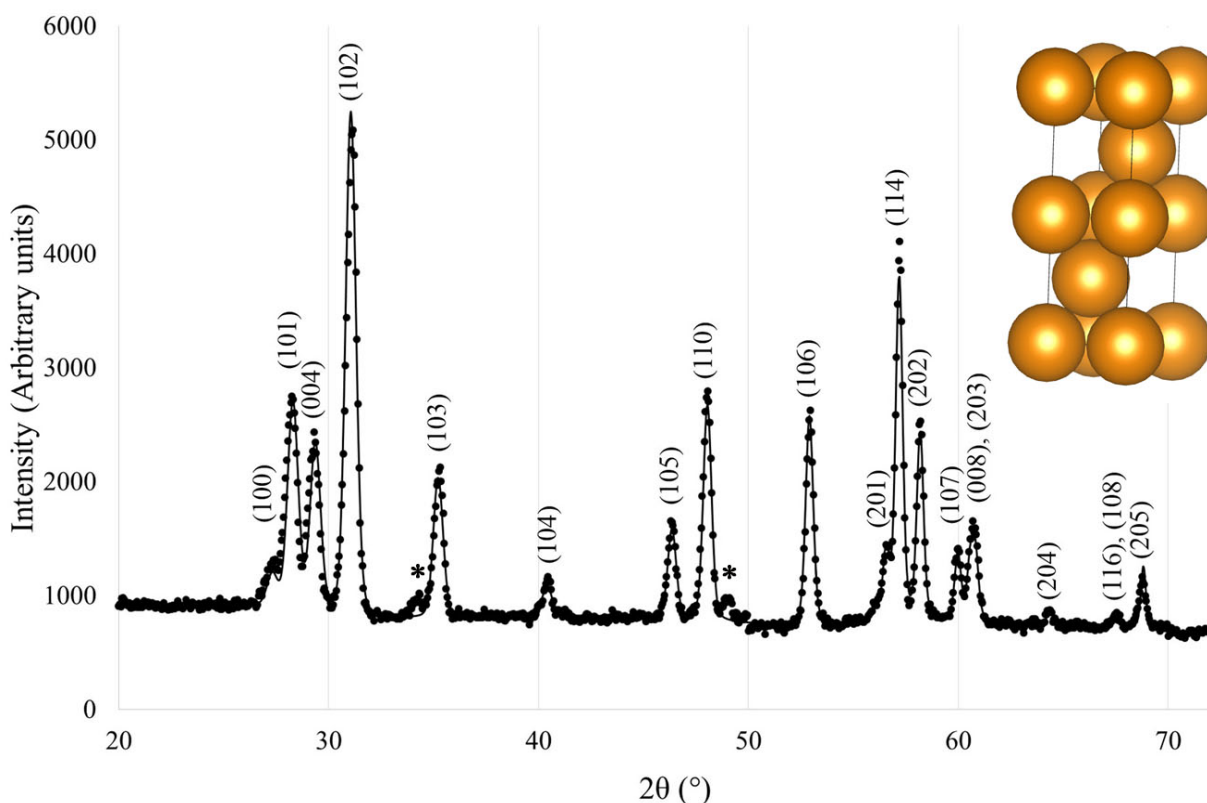
306 ^a Oxide compositions are normalized without oxygen.

307

308 **3.5 Neutron Diffraction characterization of non-irradiated samples**

309 Attempts were made to characterize the alloys with X-ray diffraction. The extremely oxophilic
 310 nature of the lanthanides prevented data collection prior to oxidation, even with coatings to prevent
 311 oxidation. Due to this, neutron diffraction, with the samples sealed in vanadium sample cans under argon,
 312 was used to characterize the alloys.

313 Figure 6 and Table 8 show the neutron diffraction results for the single phase Ln alloy refined in the
 314 hexagonal structure with space group $P6_3/mmc$, Joint Committee on Powder Diffraction Standards
 315 (JCPDS) 04-020-6110 [35]. The lattice parameters, fractional coordinates, occupancies, and thermal
 316 parameters (Uiso) are listed in Table 8. The assumption that the lanthanides were all in special positions
 317 was in good agreement with JCPDS 04-020-6110 and had a goodness-of-fit of $wRp = 0.058$ and $\chi^2 =$
 318 3.30 . The artifact peaks at 34.02° , 48.89° (marked with *) suggest a minor presence of the cubic oxide
 319 phase based CeO, $Fm-3m$ (JCPDS 04-007-4020 and 04-003-5175). An additional peak at 60.90° is not
 320 evident due to surrounding peaks.
 321



322
 323 Figure 6. Neutron diffraction pattern for Ln alloy. The inset shows the unit cell structure for this
 324 compound. All peaks correspond to a hexagonal structure.
 325

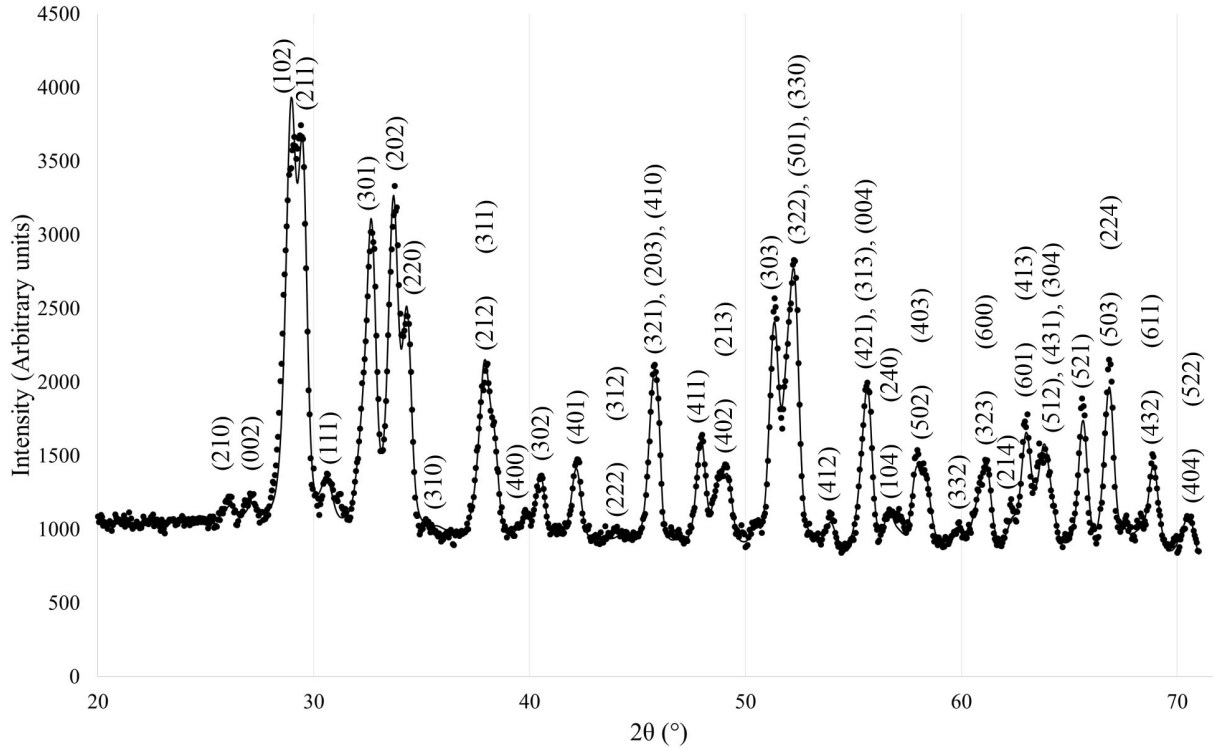
326 Table 8. Refinement results for Ln alloy in $P6_3/mmc$: RT 28M.

a (Å)	b (Å)	c (Å)	α	β	γ	Vol. (Å ³)
---------	---------	---------	----------	---------	----------	------------------------

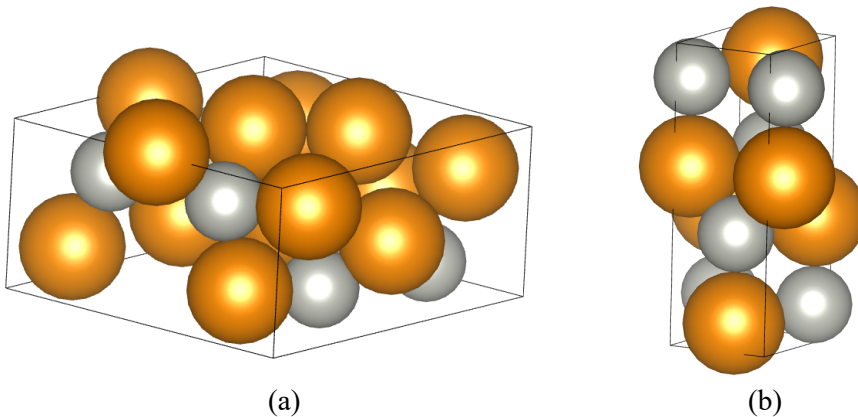
Label	Wyckoff	x	y	z	Occ.	Uiso
La	2a	0.00	0.00	0.00	0.0707(11)	0.0194(43)
Nd	2a	0.00	0.00	0.00	0.5311(15)	0.0163(80)
Pr	2a	0.00	0.00	0.00	0.1661(11)	0.0229(84)
Ce	2a	0.00	0.00	0.00	0.2537(12)	0.0236(46)
La	2c	0.3333	0.6667	0.25	0.0758(74)	0.0248(40)
Nd	2c	0.3333	0.6667	0.25	0.5357(15)	0.0098(79)
Pr	2c	0.3333	0.6667	0.25	0.1667(38)	0.0199(84)
Ce	2c	0.3333	0.6667	0.25	0.2543(30)	0.0141(46)

327
328 Figures 7 and 8 and Tables 9 and 10 show the neutron diffraction results for the Ln-Pd alloy,
329 containing two phases. The primary phase had a Nd₇Pd₃ structure and was refined in the hexagonal
330 structure with space group *P6₃mc* (JCPDS 04-001-1116). The lattice parameters, fractional coordinates,
331 occupancies, and thermal parameters (Uiso) for space group *P6₃mc* are listed in Table 9. The secondary
332 phase had a LaPd orthorhombic structure and was refined with space group *Cmcm* (JCPDS 04-004-3283).
333 The lattice parameters, fractional coordinates, occupancies, and thermal parameters (Uiso) for space
334 group *Cmcm* are listed in Table 10. The assumption that the lanthanides had the same atomic positions
335 showed a goodness-of-fit of wRp = 0.053 and Chi² = 3.40. Peaks for lanthanide oxides, such as the
336 artifact peaks observed in the Ln alloy, shown in Figure 6, for CeO, or peaks for Nd₂O₃ (28.99°, 38.19°,
337 and 53.52°, trigonal *P321* Nd₂O₃, JCPDS 04-015-5004) are likely present, but are not observed due to
338 peak overlap. The same arc melted pin produced the neutron diffraction sample as well as the sample for
339 SEM analysis, shown in Figure 5, so an intergranular oxide is present in the sample, but not in high
340 enough concentration to observe through the Ln-Pd peaks.

341



342
 343 Figure 7. Neutron diffraction pattern for Ln-Pd alloy. All peaks correspond to a hexagonal structure
 344 except for (111) and (240), which are orthorhombic.
 345



346
 347 (a) (b)
 348 Figure 8. Illustrations for primary and secondary unit cell structures for Ln-Pd alloy. (a) unit cell from
 349 hexagonal $P6_3mc$ Ln_7Pd_3 phase, and (b) unit cell from orthorhombic $Cmcm$ LnPd phase. Silver spheres
 350 represent Pd, orange spheres represent Ln.
 351

352 Table 9. Primary phase refinement results for Ln-Pd alloy in $P6_3/mmc$: RT 28M.

a (Å)	b (Å)	c (Å)	α	β	γ	Vol. (Å ³)
10.1683(7)	10.1683(7)	6.3996(4)	90°	90°	120°	573.04(8)
Label	Wyckoff	x	y	z	Occ.	Uiso
La	2b	0.3333	0.6667	0.0410(5)	0.0384(12)	0.0121(19)
Ce	2b	0.3333	0.6667	0.0410(6)	0.1276(84)	0.0120(21)
Pr	2b	0.3333	0.6667	0.0410(5)	0.0915(67)	0.0121(11)

Nd	2b	0.3333	0.6667	0.0410(6)	0.2636(54)	0.0099(19)
La	6c	0.1224(5)	0.8776(7)	0.25	0.0405(29)	0.0255(43)
Ce	6c	0.1224(3)	0.8776(4)	0.25	0.1267(16)	0.0255(42)
Pr	6c	0.1224(3)	0.8776(3)	0.25	0.0963(10)	0.0255(45)
Nd	6c	0.1224(4)	0.8776(5)	0.25	0.2456(53)	0.0238(48)
La	6c	0.5341(6)	0.4659(7)	0.0366(5)	0.0387(16)	0.0191(22)
Ce	6c	0.5341(5)	0.4659(3)	0.0366(7)	0.1310(47)	0.0190(16)
Pr	6c	0.5341(7)	0.4659(5)	0.0366(5)	0.0954(12)	0.0191(23)
Nd	6c	0.5341(6)	0.4659(2)	0.0366(4)	0.2641(54)	0.0182(18)
Pd	6c	0.8090(5)	0.1910(6)	0.2947(3)	0.4768(67)	0.0176(28)

353

354 Table 10. Secondary phase refinement results for Ln-Pd alloy in *Cmcm*: RT 28M.

a (Å)	b (Å)	c (Å)	α	β	γ	Vol. (Å ³)
3.8330(35)	10.8153(11)	4.6222(44)	90°	90°	90°	191.61(32)
Label	Wyckoff	x	y	z	Occ.	Uiso
La	4c	0.00	0.4211(3)	0.25	0.0360(23)	0.0014(13)
Ce	4c	0.00	0.4211(2)	0.25	0.1502(40)	0.0242(23)
Pr	4c	0.00	0.4211(2)	0.25	0.0508(19)	0.0265(18)
Nd	4c	0.00	0.4211(4)	0.25	0.2142(60)	0.0320(40)
Pd	4c	0.00	0.1287(2)	0.25	0.4042(43)	0.0242(26)

355

356 4. Discussion

357 In all three irradiated fuel pins, Nd is found to be the dominant lanthanide element followed by Ce,
358 making up over 70% atomically of the fission product lanthanides as would be expected from known
359 fission yields. Most importantly, the formation of low melting intermetallic compounds between Fe and
360 Ce is a primary reason for cladding wastage, limiting the maximum operating temperature of high burnup
361 U-Zr metallic fuels. The expected yields for Pd, Nd, Ce, Pr, and La for the FFTF irradiated and ATR
362 irradiated U-10Zr fuel pins are listed in Table 1. Based on the EBR-II chemical analysis [13] and the
363 expected elemental atomic concentrations for the annular and the solid fuel pins, the lanthanides are
364 produced in much higher abundance than fission-product Pd. This was expected, since the lanthanides are
365 produced by fission in a higher yield than Pd regardless of the fissioning actinide or the neutron energy
366 spectrum in a nuclear reactor. Due to these atomic concentrations, the composition of the Ln-Pd
367 precipitates observed in the MFF-3 irradiated U-10Zr fuel pin are lanthanide rich, with Pd being ~31 at.
368 %, Table 6. This composition will primarily produce Ln₇Pd₃, with a small amount of LnPd, based on the
369 Nd-Pd phase diagram [36]. Note: The Nd-Pd binary phase diagram is the most complete diagram of the
370 constituent Ln-Pd diagrams, and will be used for phase discussions. In the annular U-10Zr fuel pin
371 irradiated in ATR, no Pd was observed in the Ln precipitates. Table 1 indicates only slightly less Pd will
372 be produced as in the MFF-3 irradiated pin. A thorough search of the annular fuel pin may reveal Ln-Pd
373 precipitates, but in this investigation, none of the lanthanide nodules analyzed contained Pd.

374 The lanthanide deposits in the outer area of the annular U-10Zr fuel, i.e. the radial region containing
375 the red box in Figure 1a, occur at the periphery of pores, and are present as nodules, as indicated with red
376 arrows in Figure 1c. This may support the proposed liquid like transport of lanthanides through the
377 interconnected porosity that forms during irradiation [3][13][37][38][39]. Typically, this would occur due
378 to either sodium or a sodium-cesium mix present in the pores, but as this is a sodium-free fuel, the liquid-
379 like transport mechanism can still proceed due to fission product cesium in the pores. During sample
380 preparation methods, methods were not employed to preserve Cs, thus it was washed away during the
381 cutting or polishing step. Barium, a decay product of Cs, is observed, though, and is co-located with the
382 lanthanide nodules in the pores. This is shown in the elemental map for Ba in Figure 2. Having Cs present
383 without Na may be beneficial for transport, though, since the lanthanides have a higher solubility in Cs at
384 reactor temperature than in Na [37-39]. The solubility is roughly 5 at. % at 600 °C based on the Cs-Pr
385 phase diagram [40], which is the only available lanthanide-Cs phase diagram for the constituents in the
386 Ln mix. No comparable phase diagrams are available for the lanthanide-Na systems. The lanthanides are
387 not soluble at room temperature, though, which accounts for the precipitation around the pore periphery.

388 The composition of the Ln-Pd precipitates observed in the MFF-3 irradiated U-10Zr fuel pin was
389 the basis for the out-of-pile experiments described herein. In previous out-of-pile investigations with Pd
390 as a minor additive to bind the lanthanides and control FCCI [13][15-17], enough Pd was added to bind
391 the lanthanides in a 1:1 Ln-Pd compound. The rationale for this amount of Pd is discussed in Ref. [13],
392 while the later studies maintained this Pd:Ln ratio for consistency [15-17]. This does not indicate a 1:1
393 compound is required for Pd to bind the lanthanides, considering the Ln₇Pd₃ compound formed in the
394 irradiated U-10Zr fuel pin. Just based on the amount of lanthanides captured per Pd atom, the Ln₇Pd₃
395 compound is the most efficient use of Pd, although this compound has the lowest solidus temperature
396 (620°C) of any of the possible intermetallic compounds, based on the Nd-Pd phase diagram [36].
397 Depending on the radial location of this compound in a fuel during irradiation, this phase may be a liquid.
398 Providing excess Pd also has drawbacks, although the solidus temperatures are much higher than Ln₇Pd₃.
399 Prior to reacting with fission product lanthanides, the presence of more Pd initially will form more PdZr₂,
400 requiring more Zr to be added to the fuel in order to maintain the performance of U-10Zr. As the fuel is
401 taken to higher burnup, this becomes a substantial amount of non-fissionable material in the fuel. These
402 are some of the factors involved in determining how much Pd should be added to the fuel, although an
403 ideal ratio of Pd to lanthanides has not been determined at this point.

404 Both phases present in the Ln-Pd alloy, LnPd and Ln₇Pd₃, are apparent in the SEM images shown
405 in Figure 4. The EDS data for these phases are very consistent, with an average composition of
406 30Nd-9Ce-7Pr-2La-52Pd at. % (LnPd) for the precipitates, and 35Nd-18Ce-10Pr-5La-32Pd at. %
407 (Ln₇Pd₃) for the bulk matrix material. At this concentration of Pd and lanthanides, the microstructure
408 should be dominated by Ln₇Pd₃ with very little LnPd. That is the case in the microstructure of the

409 irradiated fuel, shown in Figure 4. The Ln-Pd alloy is lanthanide rich, but not as rich as was intended, due
410 to the loss of lanthanides to lanthanide oxides. The presence of oxygen does not affect the phases present,
411 only the relative amount of each phase and possibly the composition of the phases. Ternary compounds of
412 Pd-Ln-O are known [41] and could form in this system, although there is no spectroscopic evidence for
413 these ternary compounds. The effect of the oxide phase is further discussed below.

414 EPMA was performed on the Ln-Pd alloy to more accurately determine the composition of each
415 phase. The lanthanide peaks in EDS have significant overlap, thus the deconvolution of peaks will affect
416 the measured composition. Measuring physical standards on the EPMA enables the quantitative
417 measurement of overlapping peaks, thus allowing a more precise peak overlap correction. In addition,
418 WDS spectroscopy has an energy resolution of about 10 eV, compared to approximately 125 eV using the
419 SDD-EDS detector employed by the SEM. Due to the energy resolution of WDS spectroscopy, peak
420 overlap is minimized by using the EPMA. The average EPMA composition obtained for Ln₇Pd₃ is
421 37.1Nd-17.3Ce-11.4Pr-4.4La-29.9Pd at. %, Table 7. Comparing to the SEM EDS measured compositions
422 listed in Table 6, all of the elements are relatively close, within ~ 2 at. %. The average EPMA
423 composition obtained for LnPd is 31.4Nd-9.1Ce-7.7Pr-2.2La-49.6Pd at. %. The SEM EDS values are
424 again within ~2 at. % of the EPMA values. These results indicate the deconvolution of the lanthanide
425 peaks in SEM-EDS is very good.

426 Normalizing the EPMA measured average compositions (Table 7) of the lanthanides in Ln₇Pd₃
427 without Pd yields 52.8Nd-24.6Ce-16.2Pr-6.3La at. %, while the normalized composition in LnPd without
428 Pd is 62.3Nd-18.1Ce-15.3Pr-4.4La at. %. Normalizing in this way does not take mass balance effects in
429 the melt into account, but allows the percentage of each lanthanide present in the phases to be determined
430 for comparison. The lanthanide composition in Ln₇Pd₃ is essentially identical to the lanthanide target
431 composition of 52.3Nd-25.4Ce-16.2Pr-6.1La at. %. This is not the case in LnPd, though, which is high in
432 Nd by 10.0 at. %, and low in Ce (by 7.3 at. %), Pr (by 0.9 at. %) and La (by 1.7 at. %). Taken together,
433 the overall composition of the lanthanides is high in Nd and low in Ce. This discrepancy in the mass
434 balance can be accounted for in the intergranular oxides. The ratio of lanthanides measured with the
435 EPMA in the oxides (removing oxygen and ~2 at. % Pd from the measurements) is
436 38.6Nd-36.6Ce-16.7Pr-8.1La at. %, so is low in Nd and high in Ce. In the cluster analysis performed on
437 the images collected in the EPMA, 9% of the surface was the intergranular lanthanide oxide phase, so the
438 amount of lanthanides present in the oxide phase on this surface is 3.5Nd-3.3Ce-1.5Pr-0.7La at. %. The
439 LnPd phase makes up 12% of the surface, with a normalized (removing Pd) Ln concentration of
440 62.3Nd-18.1Ce-15.3Pr-4.4La at. %, making the Ln concentration in LnPd 7.5Nd-2.2Ce-1.8Pr-0.5La at.
441 %. With the oxide being 9% and the LnPd phase being 12%, Nd being high and Ce being low could be
442 attributed to the presence of the oxide phase. This deviation from the target lanthanide composition has
443 also been observed in fuel alloys containing Pd as an additive, with lanthanides added to simulate fission

444 product lanthanides. For example, in U-10Zr-3.86Pd-4.7Ln wt. % [16], the normalized Ln content in
445 LnPd is 59Nd-17Ce-17Pr-7La at. % and the normalized Ln content in Ln₇Pd₃ is 47Nd-26Ce-17Pr-10La
446 at. %. In a different Pd-containing alloy, U-15Zr-1.6Pd-2.15Ln wt. % [13], the normalized lanthanide
447 composition in the LnPd phase is 72Nd-12Ce-10Pr-6La at. %. In both alloys, the LnPd intermetallic is
448 high in Nd and low in Ce, as observed in the EPMA measurements. These ratios are observed when there
449 is no appreciable lanthanide oxide present in the alloy, but there was oxide present in the melt during
450 fabrication. The low density lanthanide-oxides do not incorporate into the U-Zr alloys, but are left as
451 dross when the pin is cast. Due to this, the lower Ce content in the LnPd phase could be due to Ce being
452 preferentially pulled into the oxide phase, or the lower Ce content in the LnPd phase could be due to the
453 lanthanides in the LnPd crystal structure not being a true solid solution. There may be preference for some
454 lanthanides in the crystal structure over others, leaving the remainder as an oxide dross. Regardless of the
455 reason, the LnPd phase exhibits low Ce, while the Ln₇Pd₃ phase is close to the target lanthanide ratio.

456 In the lanthanide alloy (Ln alloy), the four lanthanides form a substitutional solid solution with
457 the hexagonal space group *P6₃/mmc*. Based on the Hume-Rothery rules, this was expected. The
458 lanthanides are very close in atomic radius, have the same covalency, have essentially the same
459 electronegativity, and all exhibit the *P6₃/mmc* crystal structure at room temperature. There is no ternary or
460 quaternary phase diagram information available, but binary phase diagrams exist for all combinations of
461 Nd, Ce, Pr, and La. Only two combinations, Ce-La [42] and Nd-Pr [43], are miscible at all compositions
462 and temperatures. The other four, Nd-Ce [44], Ce-Pr [45], Nd-La [46], and Pr-La [47], all have regions of
463 immiscibility, although they all show complete miscibility at room temperature.

464 The measured lattice parameters from neutron diffraction for the Ln alloy are $a = 3.6774(4) \text{ \AA}$ and
465 $c = 11.8563(5) \text{ \AA}$ (Table 8 in Section 3.5). Being a solid solution, the lattice parameters were evaluated to
466 determine if this alloy follows Vegard's Law. Using the weighted fraction of room temperature hexagonal
467 lattice constants [48][49] for each of the four elements in 52.3Nd-25.4Ce-16.2Pr-6.1La at. %, the lattice
468 constants of the solid solution should be $a = 3.6710 \text{ \AA}$ and $c = 11.8280 \text{ \AA}$. There is excellent agreement
469 between the calculated and measured lattice parameters, with the difference being less than 1% for both a
470 and c , indicating the alloy does follow Vegard's Law. In the irradiated annular U-10Zr fuel, the calculated
471 lattice parameters based on the interplanar distance are $a = 3.89 \text{ \AA}$ and $c = 12.55 \text{ \AA}$ (Section 3.1). This is a
472 6% expansion in each of the parameters, a and c , so is very good agreement considering the differences in
473 these samples. The out-of-pile neutron diffraction was performed on a representative lanthanide
474 composition, but it does not include other fission products. The EDS analysis listed in Table 4 shows Y,
475 Zr, Sm, and a trace amount of U are also present in the irradiated sample. Zr has essentially no solubility
476 in any of the measured elements, and is likely present as sub-nanometer precipitates, below the detection
477 limit of the instrument. The deviation in lattice parameters may be caused by the error associated with
478 measuring lattice parameters using diffraction analysis performed on a TEM. A small amount of error can

479 be introduced by the instrument setup and calibration, and due to any strain in the sample lamella.

480

481 **5. Conclusions**

482 This investigation is the characterization of the Ln alloy 52.3Nd-25.4Ce-16.2Pr-6.1La at. %, Pd-
483 Ln alloy, 36.2Nd-17.6Ce-11.2Pr-4.3La-30.7Pd at. %, and Pd-La alloy 69.7La-30.3Pd at. % by neutron
484 diffraction, SEM, and EPMA, as well as the TEM characterization of a lanthanide precipitate in an
485 irradiated annular U-10Zr fuel pin. In addition, Pd-Ln precipitates in a previously characterized irradiated
486 U-10Zr MFF3 fuel pin are shown for comparison to the out-of-pile data. The following conclusions can
487 be drawn from the data presented:

- 488 • The composition of the Ln alloy used as a fission product surrogate in out-of-pile studies has been
489 confirmed in three separate U-10Zr irradiations.
- 490 • The lanthanide nodules deposited around the periphery of the pores in the irradiated annular
491 U-10Zr pin support the proposed liquid-like transport mechanism for the lanthanides moving
492 through the interconnected pores towards the fuel periphery and cladding.
- 493 • The Ln alloy is a solid solution, forming a hexagonal unit cell. Calculating the lattice parameters
494 using a weighted fraction of the lattice parameters of the constituents generates nearly identical
495 parameters as measured by neutron diffraction. The difference is less than 1% for either a or c.
496 This solid solution follows Vegard's Law with no deviation.
- 497 • The lattice parameters determined for the lanthanide precipitate in the irradiated annular U-10Zr
498 fuel are slightly larger than the out-of-pile study, but still very close, with only a 6% change in the
499 a and c parameters. This small difference could be due to error associated with determining lattice
500 parameters from the diffraction analysis performed on the TEM.
- 501 • The standard-less SEM deconvolution and quantification of the overlapping lanthanide peaks
502 compares very well with the quantification performed with EPMA using physical standards.

503

504

505 **Acknowledgments**

506 This work was supported by the U.S. Department of Energy, Office of Nuclear Energy under DOE Idaho
507 Operations Office Contract DE-AC07-05ID14517. This manuscript has also been authored in part by a
508 contractor from UT-Battelle, LLC under Contract No. DE-AC05-00OR22725 with the US Department of
509 Energy (DOE). Accordingly, the U.S. Government retains and the publisher, by accepting the article for
510 publication, acknowledges that the U.S. Government retains a nonexclusive, paid-up, irrevocable,
511 worldwide license to publish or reproduce the published form of this manuscript or allow others to do so,
512 for U.S. Government purposes.

513
514 **U.S. Department of Energy Disclaimer**
515 This information was prepared as an account of work sponsored by an agency of the U.S. Government.
516 Neither the U.S. Government nor any agency thereof, nor any of their employees, makes any warranty,
517 express or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or
518 usefulness of any information, apparatus, product, or process disclosed, or represents that its use would
519 not infringe privately owned rights. References herein to any specific commercial product, process, or
520 service by trade name, trademark, manufacturer, or otherwise, does not necessarily constitute or imply its
521 endorsement, recommendation, or favoring by the U.S. Government or any agency thereof. The views
522 and opinions of authors expressed herein do not necessarily state or reflect those of the U.S. Government
523 or any agency thereof.

524

525 **References**

- 526 [1] D.D. Keiser Jr., "Fuel cladding chemical interaction in metallic sodium fast reactor fuels: A
527 historical perspective", *J. Nucl. Mater.*, **2019**, 514, 393-397.
- 528 [2] C. Matthews, C. Unal, J. Galloway, D.D. Keiser Jr., S.L. Hayes, "Fuel-Cladding Chemical
529 Interaction in U-Pu-Zr Metallic Fuels: A Critical Review", *Nucl. Technol.*, **2017**, 198, 231-259.
- 530 [3] Y. Xie, J. Zhang, X. Li, J.P. Isler, M.T. Benson, R.D. Mariani, C. Unal, "Lanthanide Migration
531 and Immobilization in Metallic Fuels", *Prog. in Nucl. Ener.*, **2018**, 109, 233-238.
- 532 [4] S.D. Herrmann, S. Li, *Nucl. Tech.*, **2010**, 171, 247-265.
- 533 [5] G.L. Hofman, L.C. Walters, T.H. Bauer, Metallic fast reactor fuels, *Prog. Nucl. Energy*, **1997**, 31,
534 83-110.
- 535 [6] H.J. Ryu, B.O. Lee, S.J. Oh, J.H. Kim, C.B. Lee, "Performance of FCCI barrier foils for U-Zr-X
536 metallic fuel", *J. Nucl. Mater.*, **2009**, 392, 206-212.
- 537 [7] J.H. Kim, H.J. Ryu, J.H. Baek, S.J. Oh, B.O. Lee, C.B. Lee, Y.S. Yoon, "Performance of a
538 diffusion barrier under a fuel-clad chemical interaction (FCCI)", *J. Nucl. Mater.*, **2009**, 394,
539 144-150.
- 540 [8] V. Firouzdor, J. Brechtel, L. Wilson, B. Semerau, K. Sridharan, T.R. Allen, "Development of
541 titanium diffusion barrier coatings for mitigation of fuel-cladding chemical interactions", *Surf.*
542 *Coat. Technol.*, **2013**, 219, 59-68.
- 543 [9] M.T. Benson, J.A. King, R.D. Mariani, M.C. Marshall, "SEM characterization of two advanced
544 fuel alloys: U-10Zr-4.3Sn and U-10Zr-4.3Sn-4.7Ln," *J. Nucl. Mater.*, **2017**, 494, 334-341.
- 545 [10] Y. Xie, M.T. Benson, J.A. King, R.D. Mariani, J. Zhang, "Characterization of U-Zr fuel with
546 alloying additive Sb for immobilizing fission product lanthanides," *J. Nucl. Mater.*, **2018**, 498,
547 332-340.
- 548 [11] M.T. Benson, Y. Xie, J.A. King, K.R. Tolman, R.D. Mariani, I. Charit, J. Zhang, M.P. Short, S.
549 Choudhury, R. Khanal, N. Jerred, "Characterization of U-10Zr-2Sn-2Sb and
550 U-10Zr-2Sn-2Sb-4Ln to assess Sn+Sb as a mixed additive system to bind lanthanides", *J. Nucl.*
551 *Mater.*, **2018**, 510, 210-218.
- 552 [12] R. Khanal, N. Jerred, M.T. Benson, D.A. Andersson, R.D. Mariani, I. Charit, S. Choudhury, "A
553 Novel Approach to Selection of Dopant to Immobilize Neodymium in Uranium-Based Metallic
554 Fuels", *J. Nucl. Mater.*, **2020**, 529, 151922.
- 555 [13] R.D. Mariani, D.L. Porter, T.P. O'Holleran, S.L. Hayes, J.R. Kennedy, "Lanthanides in metallic
556 nuclear fuels: Their behavior and methods for their control," *J. Nucl. Mater.*, **2011**, 419, 263-271.
- 557 [14] J.M. Harp, H.J.M. Chichester, L. Capriotti, "Postirradiation examination results of several
558 metallic fuel alloys and forms from low burnup AFC irradiations," *J. Nucl. Mater.*, **2018**, 509,

- 559 377-391.
- 560 [15] M.T. Benson, L. He, J.A. King, R.D. Mariani, "Microstructural characterization of annealed
561 U-12Zr-4Pd and U-12Zr-4Pd-5Ln: Investigating Pd as a metallic fuel additive," *J. Nucl. Mater.*,
562 **2018**, *502*, 106-112.
- 563 [16] M.T. Benson, L. He, J.A. King, R.D. Mariani, A.J. Winston, J.W. Madden, "Microstructural
564 characterization of as-cast U-20Pu-10Zr-3.86Pd and U-20Pu-10Zr-3.86Pd-4.3Ln," *J. Nucl.*
565 *Mater.*, **2018**, *508*, 310–318.
- 566 [17] M.T. Benson, Y. Xie, L. He, K.R. Tolman, J.A. King, J.M. Harp, R.D. Mariani, B.J. Hernandez,
567 D.J. Murray, B.D. Miller, "Microstructural characterization of annealed U-20Pu-10Zr-3.86Pd and
568 U-20Pu-10Zr-3.86Pd-4.3Ln", *J. Nucl. Mater.*, **2019**, *518*, 287-297.
- 569 [18] J. M. Harp, D. L. Porter, B. D. Miller, T. L. Trowbridge, W. J. Carmack, "Scanning electron
570 microscopy examination of a Fast Flux Test Facility irradiated U-10Zr fuel cross section clad
571 with HT-9," *J. Nucl. Mater.*, **2017**, *494*, 227–239.
- 572 [19] J.M. Harp, L. Capriotti, D.L. Porter, J.I. Cole, "U-10Zr and U-5Fs: Fuel/cladding chemical
573 interaction behavior differences," *J. Nucl. Mater.*, **2020**, *528*, 151840.
- 574 [20] L. Capriotti, J.M. Harp, "Characterization of a minor actinides bearing metallic fuel pin irradiated
575 in EBR-II," *J. Nucl. Mater.*, **2020**, *539*, 152279.
- 576 [21] K.E. Wright, J.M. Harp, L. Capriotti, "Electron probe microanalysis of irradiated FUTURIX-FTA
577 U-Pu-Zr alloy with added minor actinides," *J. Nucl. Mater.*, **2019**, *526*, 151745.
- 578 [22] L. Capriotti, S. Brémier, K. Inagaki, P. Pöml, D. Papaioannou, H. Ohta, T. Ogata, V.V.
579 Rondinella, "Characterization of metallic fuel for minor actinides trans- mutation in fast reactor,"
580 *Prog. Nucl. Ener.*, **2017**, *94*, 194e201.
- 581 [23] J.M. Harp, L. Capriotti, H.J.M. Chichester, "Postirradiation Examination of FUTURIX-FTA
582 metallic alloy experiments," *J. Nucl. Mater.*, **2019**, *515*, 420e433.
- 583 [24] S. Brémier, K. Inagaki, L. Capriotti, P. Pöml, T. Ogata, H. Ohta, V.V. Rondinella, "Electron
584 probe microanalysis of a METAPHIX U-Pu-Zr metallic alloy fuel irradiated to 7.0 at. % burn-up,"
585 *J. Nucl. Mater.*, **2016**, *480*, 109-119.
- 586 [25] A.G Croff, "A User's Manual for the ORIGEN2 Computer Code," ORNL/TM-7175, Oak
587 Ridge National Laboratory, July 1980.
- 588 [26] A.L. Pitner, R.B. Baker, "Metal fuel test program in the FFTF," *J. Nucl. Mater.*, **1993**, *204*,
589 124-130.
- 590 [27] W.J. Carmack, H.M. Chichester, D.L. Porter, D.W. Wootan, "Metallography and fuel cladding
591 chemical interaction in fast flux test facility irradiated metallic U-10Zr MFF-3 and MFF-5 fuel
592 pins," *J. Nucl. Mater.*, **2016**, *473*, 167-177.
- 593 [28] C.T. Chantler, "Theoretical Form Factor, Attenuation, and Scattering Tabulation for Z=1–92
594 from E=1–10 eV to E=0.4–1.0 MeV," *J. Phys. Chem. Ref. Data*, **1995**, *24*, 71–643.
- 595 [29] J.T. Armstrong, "Quantitative analysis of silicate and oxide materials: comparison of monte carlo,
596 ZAF, and ψ (pz) procedures," *Microbeam Anal.*, **1988**, 239–246.
- 597 [30] V.D. Scott, G. Love, "Quantitative Electron-Probe Microanalysis", E. Horwood, New York,
598 1983.
- 599 [31] H.M. Reitveld, "A profile refinement method for nuclear and magnetic structures," *J. Appl.*
600 *Crystallogr.*, **1969**, *2*, 65-71.
- 601 [32] B.H. Toby, "EXPGUI, a graphical user interface for GSAS" *J. Appl. Crystallogr.*, **2001**, *34*,
602 210-213.
- 603 [33] T. Yao, L. Capriotti, X. Liu, Y. Wang, J.M. Harp, J. Gan, M.T. Benson, L. He, " α -U and ω -UZr₂
604 in Neutron Irradiated U-10Zr Annular Metallic Fuel", *J. Nucl. Mater.*, **2020**, *542*, 152536.
- 605 [34] X. Liu, L. Capriotti, T. Yao, J.M. Harp, M.T. Benson, Y. Wang, L. He, "Fuel-cladding chemical
606 interaction of a prototype annular U-10Zr fuel with HT-9 cladding", *J. Nucl. Mater.*, submitted.
- 607 [35] G.C. Che, J. Liang, Y. Yi, *Jinshu Xuebao*, **1986**, *22*, B206.
- 608 [36] H. Okamoto, "Nd-Pd (Neodymium-Palladium)," *J. Phase Equilibria*, **1992**, *13*, 220–222.
- 609 [37] X. Li, J. Zhang, C. Unal, R.D. Mariani, "Behaviors of Ce, Pr, and Nd in liquid cesium by ab initio
610 molecular dynamics simulations", *J. Appl. Phys.*, **2018**, *124*, 135102.

- 611 [38] J.P. Isler, "Interactions of lanthanides and liquid alkali metals for "liquid-like" lanthanide
612 transport in U-Zr Fuel," Master thesis (The Ohio State University, 2017).
- 613 [39] J. Isler, J. Zhang, R. Mariani, C. Unal, "Experimental solubility measurements of lanthanides in
614 liquid alkalis", *J. Nucl. Mater.*, 2017, 495, 438-441.
- 615 [40] H. Okamoto, Cs-Pr (Cesium-Praseodymium), *Binary Alloy Phase Diagrams*, II Ed., Ed. T.B.
616 Massalski, **1990**, 2, 1384-1385.
- 617 [41] K.T. Jacob, T.H. Okabe, T. Uda, Y. Waseda, "System Nd-Pd-O: Phase Diagram and
618 Thermodynamic Properties of Oxides Using a Solid-State Cell with Advanced Features", *J.*
619 *Phase Equilib.*, **1999**, 20, 553-564.
- 620 [42] K.A. Gschneidner Jr., F.W. Calderwood, Ce-La (Cerium-Lanthanum), *Binary Alloy Phase*
621 *Diagrams*, II Ed., Ed. T.B. Massalski, **1990**, 2, 1075-1077.
- 622 [43] K.A. Gschneidner Jr., F.W. Calderwood, Nd-Pr (Neodymium-Praseodymium), *Binary Alloy*
623 *Phase Diagrams*, II Ed., Ed. T.B. Massalski, **1990**, 3, 2794-2797.
- 624 [44] K.A. Gschneidner Jr., F.W. Calderwood, Ce-Nd (Cerium-Neodymium), *Binary Alloy Phase*
625 *Diagrams*, II Ed., Ed. T.B. Massalski, **1990**, 2, 1085-1087.
- 626 [45] K.A. Gschneidner Jr., F.W. Calderwood, Ce-Pr (Cerium-Praseodymium), *Binary Alloy Phase*
627 *Diagrams*, II Ed., Ed. T.B. Massalski, **1990**, 2, 1095-1098.
- 628 [46] K.A. Gschneidner Jr., F.W. Calderwood, La-Nd (Lanthanum-Neodymium), *Binary Alloy Phase*
629 *Diagrams*, II Ed., Ed. T.B. Massalski, **1990**, 3, 2405-2406.
- 630 [47] H. Okamoto, La-Pr (Lanthanum-Praseodymium), *Binary Alloy Phase Diagrams*, II Ed., Ed. T.B.
631 Massalski, **1990**, 3, 2411-2413.
- 632 [48] F.H. Spedding, A.H. Daane, K.W. Herrmann, "The Crystal Structures and Lattice Parameters of
633 High-Purity Scandium, Yttrium and the Rare Earth Metals," *Acta Crystallogr.*, **1956**, 9, 559-563.
- 634 [49] K.A. Gschneidner Jr., R.O. Elliott, R.R. McDonald, "Effects of alloying additions on the $\gamma \leftrightarrow \alpha$
635 transformation of cerium. Part I. Pure cerium," *Phys. Chem. Solids*, **1962**, 23, 555-566.
- 636