

# Mechanisms of Gas Precipitation in Plasma- Exposed Tungsten

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## Mechanisms of gas precipitation in plasma-exposed tungsten

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**Abstract** – Precipitation in subsurface bubbles is a key process that governs how hydrogen isotopes migrate through and become trapped within plasma-exposed tungsten. We describe a continuum-scale model of hydrogen diffusion in plasma-exposed materials that includes the effects of precipitation. The model can account for bubble expansion via dislocation loop punching, using an accurate equation of state to determine the internal pressure. This information is used to predict amount of hydrogen trapped by bubbles, as well as the conditions where the bubbles become saturated. In an effort to validate the underlying assumptions, we compare our results with published positron annihilation and thermal desorption spectroscopy data, as well as our own measurements using the tritium plasma experiment (TPE).

**PSI-19 keywords:** tungsten, bubbles and blisters, hydrogen inventory, diffusion

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## 32 I. Introduction

33 Hydrogen migration and trapping in plasma-exposed tungsten are complex processes, and  
34 developing models which capture the correct physical mechanisms is a challenging task. A review of  
35 the published experimental database reveals that a wide range of trapping mechanisms have been  
36 previously observed [1]. Of particular interest is precipitation, where atomic hydrogen in solution  
37 recombines into H<sub>2</sub>(g) upon encountering sub-surface voids and bubbles [2]. These precipitates have  
38 the capacity to trap a large amount of hydrogen at shallow depths. Upon mechanical failure of the  
39 bubble structure (typically through fissures leading to the surface) the trapped hydrogen gas is able to  
40 escape [3]. This has the benefit of reducing the total retention in plasma facing materials by  
41 preventing diffusion to deeper trap sites.

42 Being able to more rigorously predict the amount of hydrogen trapped in the bubbles would be  
43 particularly helpful for estimating tritium inventory in ITER. The broad distribution of bubble sizes  
44 possible with different material microstructures and plasma conditions (e.g. exposure to mixtures of  
45 both He and H) introduces challenges to predicting how efficiently the bubbles trap hydrogen under  
46 different conditions. These processes are being addressed by a number of researchers using density  
47 functional theory (DFT), molecular dynamics (MD), and kinetic Monte Carlo approaches [4]. Such  
48 models provide a wealth of fundamental insight into basic physical processes, but tend to be restricted  
49 to small spatial domains over very limited time scales. A number of general-purpose continuum-scale  
50 diffusion codes, such as TMAP7 [5], are in wide use within the magnetic fusion community and have  
51 been applied to modeling thermal desorption spectroscopy from plasma-exposed materials. These  
52 models allow point defects to be simulated, but cannot yet account for precipitation. Hence, care must  
53 be taken when applying them to situations where the effect of bubbles is important.

54 Incorporating approximations to atomic-scale effects into a simpler model offers a pathway to  
55 simulate trapping in bubbles for practical experimental conditions. In a previous study [6], we  
56 developed the basic theoretical framework needed to model precipitation in tungsten at a continuum-  
57 scale, extensively leveraging Cowgill's prior models of <sup>3</sup>He bubble growth in metal tritides [7]. Using  
58 this approach, we were able to identify the conditions amenable to hydrogen precipitation using the

assumption that the bubbles expand by dislocation loop punching. In this article, we describe several extensive modifications to our model, including an improved treatment of trapping by bubbles. In an effort to refine and validate the model, we compare simulated test cases with analytical calculations, as well as published positron annihilation measurements. We also discuss initial results from our experimental work, including exposures of ITER-grade and warm-rolled tungsten sample materials using the tritium plasma experiment (TPE).

## II. Continuum-scale model and underlying assumptions

From the perspective of hydrogen precipitation in tungsten, the most important parameter is the enthalpy of solution,  $H_s$ , which is 1.04 eV relative to  $\frac{1}{2}$  H<sub>2</sub>(g) [8]. This indicates that introducing hydrogen into solution sites within tungsten is a strongly endothermic process. Thus, under most conditions hydrogen migrating through the lattice could achieve a lower energy state by recombining upon encountering a void. We developed our continuum-scale model with the objective of simulating controlled laboratory experiments, keeping in mind that any simplifications can later be relaxed to address a more realistic environment. Many of the major underlying assumptions are described in Ref. [6], and only a summary is presented here. We consider 1-D hydrogen diffusion with a uniform temperature distribution,  $T$ . The simplified governing equation that includes trapping and precipitation is

$$\partial u(x, t) / \partial t = D(T(t)) \partial^2 u(x, t) / \partial x^2 - q_T(x, t) - q_B(x, t) + \Phi(x, t)$$

where  $u(x, t)$  is the concentration of hydrogen in solution, and  $D$  is the diffusivity. The flow of hydrogen into and out of saturable traps and bubbles is dictated by  $q_T(x, t)$  and  $q_B(x, t)$  respectively, whereas the implanted flux is indicated by  $\Phi(x, t)$ . Note that because we assume that  $\partial T / \partial x = 0$ , concentration gradients arising from the Ludwig-Soret effect may be neglected. We also ignore strain effects on hydrogen diffusion.

During ion implantation, H is injected directly into solution sites within the lattice. A key consideration is the surface boundary condition which governs how the implanted hydrogen leaves the surface. Recombination coefficients for the W+H system have been measured by several researchers,

84 but the spread in these values is 10 orders of magnitude [1]. Likely contributing to this is the effect of  
 85 adsorbed impurities, which even at small concentrations decrease the recombination rate drastically.  
 86 Experimental studies that are successful in controlling surface impurities report recombination  
 87 consistent with a model based on second-order kinetics [9]. When extrapolated to W, these results  
 88 suggest fast recombination, consistent with diffusion-limited release from the surface. For this reason,  
 89 we assume boundary condition of  $u(0, t) = 0$  for the front surface.

90 As discussed in further detail in Ref. [6], we treat point defects (saturable traps) separately from  
 91 bubbles. Analytical expressions for the flow of hydrogen into point defects,  $q_t(x, t) = \partial u_t(x, t)/\partial t$ ,  
 92 have been developed by several authors; we have implemented a model similar to the one used in  
 93 TMAP7. (We refer the reader to Ref. [5] for more details.) To efficiently calculate the flow of H into  
 94 bubbles, we use a widely used analytical approach developed by Ham [10], given by the following  
 95 straightforward expression:

$$q_B(x, t) = \partial u_B(x, t)/\partial t = 4\pi D(t)r_B(x, t)N_B(x)[u(x, t) - u_{eq}(x, t)]$$

96 where  $r_B$  is the bubble radius,  $u_B$  is the concentration trapped in bubbles,  $N_B$  is the number density of  
 97 bubbles within the lattice, and  $u_{eq}$  is the equilibrium concentration in solution evaluated at the bubble  
 98 surface. Note that  $u_{eq}$  may be determined by calculating the fugacity of the high-pressure gas within  
 99 the bubble using a realistic equation of state and assuming Sievert's Law is satisfied at the bubble  
 100 interface. When  $u(x, t) < u_{eq}(x, t)$  the direction of flow is out the bubble, thereby enabling the  
 101 simulation of thermal desorption by this expression.

102 Ham's elegant expression is derived by determining the solute species concentration surrounding  
 103 an array of evenly-spaced, spherical precipitates using an eigenfunction expansion. If  $r_B$  is assumed to  
 104 be much smaller than the inter-bubble spacing, only the first term of this expansion will be important,  
 105 which simplifies the resulting expression considerably. In addition, our expression for  $q_B$  also  
 106 assumes that any temporal variations in the mobile concentration far from the precipitate are small, so  
 107 that quasi-equilibrium is always satisfied. (Given the high diffusivity of hydrogen in tungsten, this is  
 108 generally not a significant problem for most conditions encountered in laboratory experiments.)

109 Calculating the amount of hydrogen accumulated in bubbles and traps involves solving coupled  
 110 parabolic PDE's. For this purpose, we used the method of lines approach, where only the spatial  
 111 derivative is discretized. For a given segment  $i$ , the system of equations becomes:

$$du_i(t)/dt = D_i \frac{u_{i-1} - 2u_i + u_{i+1}}{\Delta x_i^2} - q_{T_i}(t) - q_{B_i}(t) + \Phi_i(t)$$

112 This converts the PDE into a coupled set of ordinary differential equations which can be rearranged in  
 113 the following form:

$$G(u(t), u_T(t), u_B(t), t) = 0$$

114 To solve this system, we used the differential algebraic equation (DAE) solver DASPK developed by  
 115 Petzhold [11]. This is a more advanced version of the DASSL routine developed specifically for stiff  
 116 DAE systems. The solver iteratively attempts to minimize the value of  $G$  subject to accuracy  
 117 constraints.

### 118 **III. Chemical equilibrium and equation of state**

119 Determining whether hydrogen will flow into subsurface bubbles requires calculating the solution  
 120 concentration that will be in equilibrium the precipitated gas. One must therefore determine the  
 121 fugacity of the gas in the bubbles accurately using an equation of state that takes into account non-  
 122 ideal gas effects at high pressure. Fig. 1 shows a comparison of the existing experimental database  
 123 [12-14] and curve fits [15-16] for the behavior of hydrogen at high pressure at ambient temperature.  
 124 Note that the ideal gas law approximation diverges in the pressure range 10-100 MPa.

125 The equation of state proposed by Tkacz [16] appears to be quite accurate over a wide pressure  
 126 range:

$$v = Ap^{-1/3} + Bp^{-2/3} + Cp^{-4/3} + (D + ET)p^{-1}$$

127 Note that for H<sub>2</sub>,  $A = 176.330$ ,  $B = -633.675$ ,  $C = -304.574$ ,  $D = 731.393$ , and  $E = 8.59805$ . It  
 128 is important to keep in mind that while specific volume is easily expressed in terms of pressure, the  
 129 inverse relationship cannot be easily expressed in a simple, closed-form equation. In addition, because

Tkacz's equation uses a polynomial fitting function, it provides erratic results far outside of its intended range of use. This is not immediately obvious from Fig. 1, and therefore care must be taken when applying this fit. For this reason, we transition to the equation of state developed by San Marchi et al. [15] at intermediate pressures, as it asymptotes to the ideal gas law.

In small hydrogen bubbles, pressures  $> 1$  GPa are needed for dislocation loop punching. The magnitude of this pressure is rather striking, particularly because this is near the point where hydrogen forms a molecular solid. (Early experiments with diamond anvil high-pressure cells found solidification at 5.7 GPa and 25 °C; later x-ray measurements confirmed the structure to be hexagonal-close-packed with a  $H_2$  molecule at each lattice point [14].) This highlights the extreme conditions encountered in plasma-exposed materials, but causes one to question whether the pressure in hydrogen bubbles in tungsten could realistically grow to such high levels. We point out that such pressures are not unprecedented, particularly given prior observations of  $^3He$  solidification in  $PdT_x$  with nuclear-magnetic-resonance (NMR) analysis [17]. Because it is straightforward to incorporate into a continuum model, we will focus primarily on the dislocation loop-punching mechanism described above.

#### IV. Results and Discussion

To test the underlying assumptions incorporated into our simulations, we considered a series of thermal desorption spectroscopy (TDS) and positron annihilation (PA) experiments by Van Veen et al. [18]. To summarize their work, W(100) single crystals were initially damaged with 6 MeV protons. The beam energy was adjusted to produce a uniform concentration of vacancies ( $4 \times 10^{-5}$  /W) up to a depth of 35  $\mu m$ . After heating to 1200 K for 15 min., voids formed throughout this volume at a concentration of  $10^{-6}$  /W. The average void diameter corresponded to approx. 1 nm ( $\sim 30 - 50$  vacancies, as determined by PA.) Afterward, the samples were bombarded with 2 keV  $H_2^+$  with an incident flux of  $\Phi = 1.25 \times 10^{18} \text{ m}^{-2} \text{ s}^{-1}$ ; the authors considered doses of  $10^{22} \text{ m}^{-2}$  and  $10^{23} \text{ m}^{-2}$ . All exposures were performed at 350 K.

Many experimental studies propose a model for the energetics that includes the existence of point defects such as vacancies ( $H_t=1.4$  eV), and surface traps corresponding to the dissociation enthalpy of hydrogen chemisorbed on the interior wall of a void (1.8-2.1 eV) [1]. The ability of our model to predict the equilibrium fraction of the traps filled as a function of temperature is therefore crucial. As illustrated in Fig. 2(a) for 1.4 eV traps, our model is in accord with an analytical expression from Ref. [2]. It is also useful to consider the special case where the concentration of traps is much larger than the mobile concentration at the depth of implantation, i.e.  $N_t \gg u(r, t)$ , where  $r$  is the range of the implanted ions. Under these conditions, the traps will be completely filled up to an interface defined by  $x = \sqrt{2Dtu(r, t)/N_t}$ , and empty thereafter. (We refer the reader to Ref. [2] for more details.) Through a rearrangement of this expression, it is straightforward to calculate the total retention as a function of dose. As we will describe below, such a model can also be readily adapted to simulating the filling of voids, providing an analytical check for our model. To simulate 2 keV  $H_2^+$  (or 1 keV  $H^+$ ) ion bombardment, we calculated an implantation range of 9 nm using TRIM [19]. Consistent with the PA measurements, we assume that the vacancies produced by the 6 MeV proton irradiation became mobile and combined to form voids during the 1200 K anneal. With this in mind, we populated a region within 35  $\mu\text{m}$  of the surface uniformly with voids.

Consistent with the findings of Van Veen et al., we found that for the ion fluxes described in Ref. [18], 1 nm dia. voids were not able to accumulate enough hydrogen to punch dislocation loops. Instead, equilibrium with the hydrogen in solution was achieved for 40 H/bubble. This is somewhat less than the  $\sim 100$ -200 H/bubble determined by Van Veen et al. from their PA measurements. We suspect this is partially due to their reliance on an older equation of state for high pressure H when fitting their PA measurements. In any case, a slight adjustment to the average void diameter (to 1.5 nm) brings the number of hydrogen that can be accommodated at saturation to 150 H/bubble, which is very much in accord with their measurements. To assess the total amount of hydrogen trapped within the voids, Van Veen et al. used thermal desorption spectroscopy. For doses of  $10^{22} \text{ m}^{-2}$  and  $10^{23} \text{ m}^{-2}$ , the retention was determined to be  $1.2 \times 10^{20} \text{ m}^{-2}$  and  $2.0 \times 10^{20} \text{ m}^{-2}$ , respectively. We were able to



181 obtain comparable values by adjusting the void concentration to 2 appm. This is certainly within the  
182 uncertainty of the PA measurements.

183 Under the conditions described above, the voids behave essentially like saturable traps. We  
184 should point out that it is conceivable that one could match the aforementioned retention results by  
185 simply assuming an evenly dispersed population of point defects with dissociation energies ranging  
186 between of 1.8-2.1 eV, as illustrated in Fig. 2(b). However, the model we have proposed here presents  
187 a much more accurate depiction of the actual physical processes involved with trapping within  
188 bubbles. As illustrated in Fig. 2(c), the voids are completely filled up to an interface and then empty  
189 afterward, consistent with the behavior described previously.

190 These initial results certainly provide evidence that a continuum-scale approach could be applied  
191 to accurately modeling trapping by bubbles in tungsten. However several key obstacles still remain.  
192 The next step in developing this model is applying it to the growth of large precipitates in more  
193 realistic laboratory conditions and using this information to eventually predict total tritium inventory  
194 in ITER. A key challenge associated with this will be determining how to include the effects of the  
195 micro-scale structure of the material into the model. To illustrate the effect of material processing,  
196 consider Fig. 3 which shows a comparison of post-mortem surface morphology between ITER-grade  
197 (A.L.M.T. Corp.) and warm-rolled (PLANSEE) tungsten samples. Both materials were exposed to  
198 100 eV  $D_2^+$  ions in the tritium plasma experiment (TPE) located at Idaho National Laboratory. (Please  
199 see Ref. [20] for a more comprehensive description of the experimental hardware.) In each case, the  
200 samples were electrical discharge machined from the raw material, polished to an RMS surface  
201 roughness of 0.2  $\mu m$ , and then stress relieved in ultra-high vacuum at 1000 °C for 1 hr. The exposure  
202 conditions for both samples were nearly identical, as indicated in Fig. 3. Nevertheless, the two  
203 surfaces have drastically different surface morphologies. Whereas the ITER tungsten is characterized  
204 by small surface bubbles < 5  $\mu m$  in dia., the warm-rolled tungsten contains blisters in excess of 50  $\mu m$ .  
205 In the case of the warm-rolled tungsten, the grains are elongated parallel to the surface, providing a  
206 well-defined boundary for subsurface cracks to form [6]. In the case of the ITER-grade samples, the

207 grain orientation is perpendicular to the surface, which is not conducive to such large-scale  
208 delaminations.

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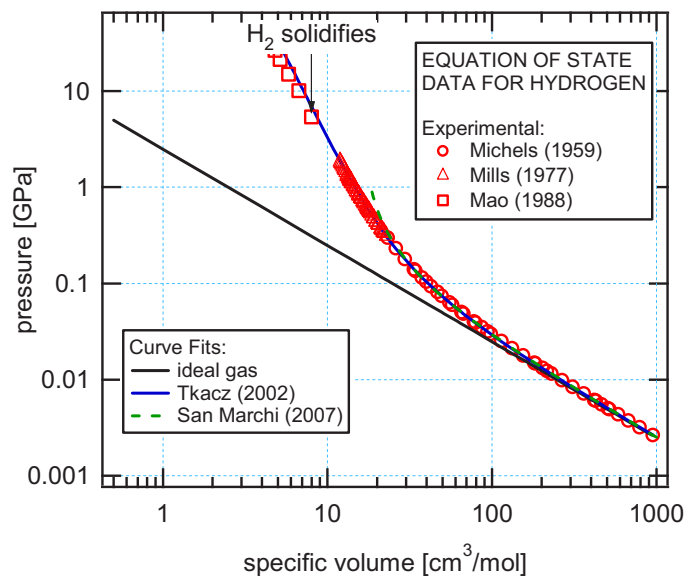
244 **Figure Captions**

245 Fig. 1: Comparison between empirical hydrogen equation of state and experimental measurements.

246 Fig. 2: Comparison of bubble model trapping results with calculations and experiments: (a)  
 247 temperature variation of fraction of traps filled, (b) retention in 1.5 nm dia. voids as a function of  
 248 fluence, and (c) concentration contained in voids vs. depth for the experiments of Van Veen et al. [18].  
 249 In panel (b), the total retention measured by Van Veen et al. for fluences of  $10^{22} \text{ m}^{-2}$  and  $10^{23} \text{ m}^{-2}$  is  
 250 indicated by the solid markers.

251 Fig. 3: Surface morphology of (a) ITER-grade and (b) warm-rolled tungsten samples exposed to 100  
 252 eV  $\text{D}_2^+$ . [Irradiation conditions: (a)  $T_{surf}=350 \text{ }^\circ\text{C}$ ;  $\Phi=1.5 \times 10^{22} \text{ m}^{-2}\text{s}^{-1}$ ;  $F=1.1 \times 10^{26} \text{ m}^{-2}$ ; (b)  
 253  $T_{surf}=385 \text{ }^\circ\text{C}$ ;  $\Phi=1.1 \times 10^{22} \text{ m}^{-2}\text{s}^{-1}$ ;  $F=0.8 \times 10^{26} \text{ m}^{-2}$ ]

254

255 **Figures**

256

257 Fig. 1: Comparison between empirical hydrogen equation of state and experimental measurements.

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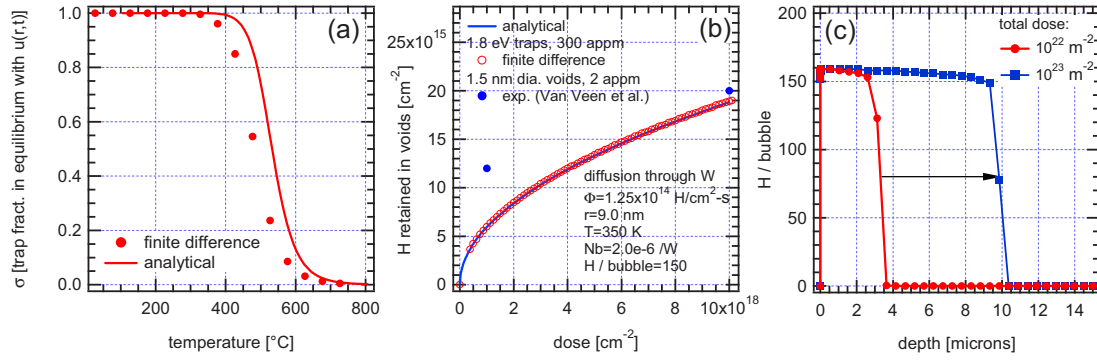
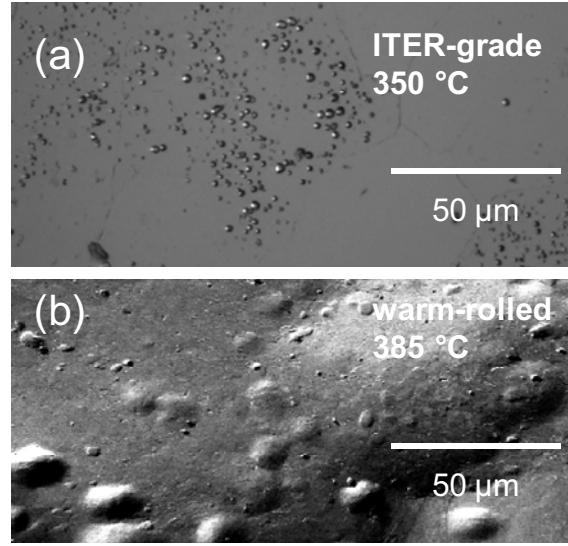


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265

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