

NEET Enhanced Micro-Pocket Fission Detector for High Temperature Reactors - FY16 Status Report

T.Unruh, D. McGregor, M. Reichenberger

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EXECUTIVE SUMMARY

A collaboration between the Idaho National Laboratory (INL), the Kansas State University (KSU), and the French Atomic Energy Agency, Commissariat à l'Énergie Atomique et aux Energies Alternatives, (CEA), has been initiated by the Nuclear Energy Enabling Technologies (NEET) Advanced Sensors and Instrumentation (ASI) program for developing and testing High Temperature Micro-Pocket Fission Detectors (HT MPFD), which are compact fission chambers capable of simultaneously measuring thermal neutron flux, fast neutron flux and temperature within a single package for temperatures up to 800 °C.

The MPFD technology utilizes a small, multi-purpose, robust, in-core fission chambers and thermocouple. As discussed within this report, the small size, variable sensitivity, and increased accuracy of the MPFD technology represent a revolutionary improvement over current methods used to support irradiations in US Material Test Reactors (MTRs).

Previous research conducted through NEET ASI¹⁻⁴ has shown that the MPFD technology could be made robust and was successfully tested in a reactor core. This new project will further the MPFD technology for higher temperature regimes and other reactor applications by developing a HT MPFD suitable for temperatures up to 800 °C.

This report summarizes the research progress for year two of this three year project. Highlights from research accomplishments include:

- Continuation of a joint collaboration between INL, KSU, and CEA. Note that CEA is participating at their own expense because of interest in this unique new sensor.
- An updated parallel wire HT MPFD design was developed.
- Program support for HT MPFD deployments was given to Accident Tolerant Fuels (ATF) and Advanced Gas-cooled Reactor (AGR) irradiation test programs.
- Quality approved materials for HT MPFD construction were procured by irradiation test programs for upcoming deployments.
- KSU performed electrical contact and fissile material plating and delivered fissile HT MPFD parts to INL for final construction of HT MPFD prototype.
- Prototype HT MPFD was constructed at INL.
- The HT MPFD has been modeled in MCNP to optimize the amount of fissile material deposition as well as location in the irradiation test.
- Detector electronic specifications have been revised and tested by KSU.
- Project meetings were held at INL and KSU to discuss the roles and responsibilities between INL, KSU, and CEA for development and deployment of the HT MPFDs.

As documented in this report, FY16 funding has allowed the project to meet year one planned accomplishments to develop a HT MPFD. In addition, the accomplishments of this project have attracted independent funding from other Department of Energy Office of Nuclear Energy (DOE-NE) programs for MTR irradiations of the MPFD technology. These are significant opportunities for this *NEET Enhanced Micro-Pocket Fission Detector for High Temperature Reactors* project because the irradiation expense of these experiments could not be included in the original project scope.

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ACRONYMS

AFC	Advanced Fuel Cycle
AGR	Advanced Gas-cooled Reactor
ART	Advanced Reactor Technologies
ASI	Advanced Sensors and Instrumentation
ATF	Accident Tolerant Fuels
ATR	Advanced Test Reactor
ATRC	Advanced Test Reactor Critical facility
BTB	Back-to-Back
BWR	Boiling Water Reactor
CEA	Commissariat à l'Énergie Atomique et aux Energies Alternatives
DOE	Department of Energy
DOE-NE	Department of Energy Office of Nuclear Energy
EDL	Electronics Design Laboratory
EIL	Energy Innovation Laboratory
EGT	Experiment Guide Tube
FCRD	Fuel Cycle Research and Development
FNDS	Fast Neutron Detection System
FY	Fiscal Year
HFIR	High-Flux Isotope Reactor
HPIL	Health Physics Instrument Laboratory
HTGR	High Temperature Gas Reactor
HTIR-TC	High Temperature Irradiation Resistant Thermocouple
HT MPFD	High Temperature Micro-Pocket Fission Detector
HTTL	High Temperature Test Laboratory
INL	Idaho National Laboratory
IRP	Integrated Research Project
KSU	Kansas State University
LWR	Light Water Reactor
LWRS	Light Water Reactor Sustainability
MPFD	Micro-Pocket Fission Detector
MTR	Material Test Reactor
NEET	Nuclear Energy Enabling Technology
NGNP	Next Generation Nuclear Plant
NIST	National Institute of Standards and Technology
NSUF	National Scientific User Facility
ORNL	Oak Ridge National Laboratory
PWR	Pressurized Water Reactor
SFR	Sodium Fast Reactor
SMART	Semiconductor Materials And Radiological Technologies

SMR	Small Modular Reactors
SPND	Self Powered Neutron Detector
TRIGA	Training Research Isotope-production General Atomics
TREAT	Transient REActor Test

NEET Enhanced Micro Pocket Fission Detector for High Temperature Reactors - FY15 Status Report

1. INTRODUCTION

Irradiation testing, which is often performed in harsh conditions (e.g., high flux, high temperature, etc.) in Materials Test Reactors (MTRs), is essential for evaluating the performance of new materials and fuels prior to use in commercial reactors. Accurate monitoring of these harsh conditions with compact sensors, that are less likely to disturb the conditions of interest, is necessary to quantify any degradation that occurs during irradiation.

Miniature fission chambers and thermocouples have been used in-core at research and test reactors throughout the world; however, none have been deployed in a single compact package to survive the harsh conditions that exist within many tests conducted in MTRs. Previous research conducted through Nuclear Energy Enabling Technologies (NEET) Advanced Sensors and Instrumentation (ASI) ¹⁻⁴ has shown that the Micro-Pocket Fission Detector (MPFD) technology could be made robust and was successfully tested in a reactor core. However, it was recognized that a High Temperature MPFD (HT MPFD) would be of interest to irradiation test campaigns that test materials at elevated temperatures. The following report summarizes FY15 activities completed to develop a HT MPFD capable of measuring thermal neutron flux, fast neutron flux, and temperature within a single compact sensor for temperatures up to 800 °C.

1.1. Motivation/Objective

During the last 40 to 50 years, various sensors have been developed to meet the needs of irradiation testing for materials and fuels in MTRs. Development of these sensors is an on-going process because they are continuously improved and refined to overcome operational shortcomings associated with more complicated irradiation testing requirements. Recent interest in testing to support new reactor designs can require higher temperatures, higher fluxes, or more corrosive test conditions. The next generation of sensors is under development that can survive these conditions.

In-core fission chamber design has remained relatively unchanged for decades. Improvements in performance, overall size, and operational modes have been implemented; however, all have been based on the same design that utilizes coaxial cylinders with a high pressure fill gas. These design considerations limit the robustness, lifetime, size, and operational performance of such sensors in high performance MTR environments.

The MPFD technology utilizes the same operational concept of existing fission chamber designs, but deploys different geometry, construction, materials and operational characteristics that are discussed further in Section 2.1.3.2. The small design also allows two or more of these neutron detectors and a thermocouple to be co-located within a single sensor sheath such that thermal flux, fast flux, and temperature can be simultaneously measured at very near the same location in the experiment with a single penetration in the experimental pressure boundary.

The MPFD technology offers the potential for improved, high accuracy, and higher fidelity data of interest to Department of Energy office of Nuclear Energy (DOE-NE) programs. Deployment of multiple compact HT MPFDs in irradiation tests will provide much needed information for current efforts to develop and deploy high-fidelity, multi-scale, multi-physics computer codes.

1.2. Objectives of NEET ASI HT MPFD Research

Several DOE-NE programs, such as the Fuel Cycle Research and Development (FCRD), Advanced Reactor Technologies (ART), Light Water Reactor Sustainability (LWRS), Small Modular Reactors (SMR), and Next Generation Nuclear Plant (NGNP) programs, are investigating new fuels and materials for advanced and existing reactors. The NEET ASI in-pile instrumentation development activities are focused upon addressing cross-cutting needs for DOE-NE irradiation testing by providing higher fidelity, real-time data with increased accuracy and resolution from smaller, compact sensors that are less intrusive.⁵ In particular, flux sensors are required for characterizing fuel performance as a function of burnup and cladding and structural material degradation as a function of neutron exposure. The NEET ASI program initiated the HT MPFD project because it addresses this cross-cutting need by developing, fabricating, and evaluating the performance of compact multi-purpose fission chambers with integral temperature sensors that could survive up to 800 °C.

A key objective of several DOE-NE programs is to understand the performance of candidate fuels and materials during irradiation. Hence, NEET research must produce sensors able to withstand the operating conditions of interest to these DOE-NE programs. It is also important that the sensors be able to measure test parameters with the desired accuracy and resolution required by these DOE-NE programs and that the sensors be compact to minimize their impact on irradiation test data. Recognizing that DOE-NE program needs change, the NEET program holds annual reviews to ensure that projects continue to meet DOE-NE program needs. Specific details related to requirements for thermal flux, fast flux, and temperature sensors requested by DOE-NE programs for fuels and material irradiations are summarized in this section.

1.2.1. Fuel

In-situ instrumentation is desired to provide real-time data on fuel performance phenomena. Currently, drop-in or static capsule experiments only allow data to be obtained at the endpoint of an experiment. Although visual examinations and material property measurements may be made on samples at the end of the test, such measurements are subject to error because of handling and because measurements are not made under prototypic pressures, fluxes, and/or temperatures. Sensors included in such tests can only provide insights about the integral neutron fluence or peak temperatures. In-situ sensors in irradiation tests with instrumentation leads can provide data showing the evolution of particular phenomena over time.

Clearly, real-time data obtained during an irradiation are advantageous. However, it is important to understand what additional sensors are needed to obtain data with the required accuracy and resolution. As part of this NEET effort, appropriate documents were reviewed and cognizant technical experts were contacted to gain insights related to the temperature, flux levels, and fluences proposed for fuel irradiation tests. Results from this review are summarized in Table 1.

1.2.2. Materials

Using a Light Water Reactor (LWR) as an example, there are many different types of materials that makeup the construction; over 25 different metal alloys can be found within the primary and secondary systems, not to mention the concrete containment vessel, instrumentation and control, and other support facilities. Over the initial forty-year lifetime of a LWR, internal structural components may expect to see up to $\sim 10^{22}$ n/cm² in a Boiling Water Reactor (BWR) and $\sim 10^{23}$ n/cm² in a Pressurized Water Reactor (PWR) ($E > 1$ MeV), corresponding to ~ 7 dpa and 70 dpa, respectively. The neutron irradiation field can produce large property and dimensional changes in materials. Such changes occur primarily via five radiation damage processes: radiation-induced hardening and embrittlement, phase instabilities from radiation-induced or radiation-enhanced segregation and precipitation, irradiation creep, volumetric swelling from cavity formation, and high temperature helium embrittlement due to formation of helium-filled cavities on grain boundaries. Extending the service life of a reactor will increase neutron fluence and suscepti-

Table 1. Summary of desired parameters for detection during fuel irradiation tests

Parameter	Representative Peak Value ^a	Desired ^a	
		Accuracy	Spatial Resolution
fuel temperature	Ceramic LWR - 1400 °C	2%	1-2 cm (axially); 0.5 cm (radially)
	Ceramic SFR - 2600 °C		
	Metallic SFR - 1100 °C		
	TRISO HTGR - 1250 °C		
cladding temperature	Ceramic LWR - <400 °C	2%	1-2 cm (axially)
	Ceramic SFR - 650 °C		
	Metallic SFR - 650 °C		
	TRISO GCFR	NA	NA
pressure in fuel rod plenum	Ceramic LWR - 5.5 MPa	5%	NA ^b
	Ceramic SFR - 8.6 MPa		
	Metallic SFR - 8.6 MPa		
	HTGR - NA		
LWR, SFR, and HTGR fission gas release (amount and composition)	0-100% of inventory	10%	NA
LWR and SFR fuel and cladding dimensions (includes fuel / cladding gap size); HTGR- NA	Initial Length, 1 cm	1%	NA
	Outer diameter/Strain, 0.5 cm/5-10%	0.1%	NA
	Fuel-Cladding Gap (0-0.1 mm)	0.1%	NA
LWR, SFR, and HTGR fuel morphology/microstructure/cracking/ constituent redistribution	Grain size, 10 µm	5%	1-10 µm
	Swelling/Porosity, 5-20%	2%	NA
	Crack formation and growth	2%	10-100 µm
fuel thermal properties	Thermal conductivity Ceramic: < 8 W/mK; Metallic: < 50 W/mK; TRISO pebble/compact: 4-12 W/mK	4%	< 1 cm (radially)
	Density (inferred from changes in length, diameter, porosity, etc.) Ceramic: < 11 g/cm ³ ; Metallic: < 50 g/cm ³ ; TRISO pebble/compact: 2.25 g/cm ^{3c}	2%	NA
real-time thermal and fast neutron flux for estimating fluence and fuel burnup for fuel irradiations	Thermal neutron flux - ~1-5 x 10 ¹⁴ n/cm ² -s	1-10%	5 cm (axially)
	Fast neutron flux (E> 1 MeV) - ~1-5x10 ¹⁴ n/cm ² -s	15%	5 cm (axially)

a. Representative peak values, accuracy, and resolution are based on engineering judgement by cognizant program experts.

b. NA-Not Applicable.

c. Value dependent upon particle packing fraction and matrix.

bility to radiation damage (although new damage mechanisms are possible). Likewise, when one considers the additional conditions proposed within DOE-NE programs for NGNP and Sodium Fast Reactor (SFR) operation, material performance becomes more complex.

Currently, most material irradiations are relying on Post Irradiation Examination (PIE) to characterize material properties after an irradiation is completed. However, enhanced in-pile instrumentation offers the potential to supplement current PIE evaluations by increased accuracy, higher fidelity data since measurements are obtained at the conditions of interest. Currently-requested accuracies and resolutions for these materials are listed in Table 2.

Table 2. Summary of desired parameters for detection during materials irradiation tests

Parameter	Representative Peak Values ^{a,b}	Desired ^{a,b}	
		Accuracy	Spatial Resolution
Material temperature distribution	LWRS Fuel Cladding - >1200 °C	2%	1-2 cm (axially); 0.5 cm (radially)
	LWRS Vessel and Internal Materials - 500 °C		
	HTGR and LWRS high temperature alloys - 950 °C		
	HTGR Graphite - 600 to 1200 °C	± 50 °C (axially) ± 40 °C (radially)	NA
Material dimensional changes due to swelling	Initial Specimen Length, HTGR Graphite - 2.54 cm	1%	NA
	Outer diameter/Strain, LWR vessel and internal materials - 0.5 cm/5-10%	0.1%	NA
Material morphology/ microstructure/cracking/ constituent redistribution	Grain size, LWR vessel and internal materials > 10 µm	5%	1-10 µm
	Swelling/Porosity, LWR vessel and internal materials 5-20%	2%	10-100 µm
	Crack formation and growth LWR vessel and internal materials > 10 µm	2%	10-100 µm
Material thermal properties	Thermal conductivity HTGR Graphite - ~80 W/m-K	4%	< 1 cm (radially)
	Thermal conductivity LWR vessel and internal materials ~50 W/m-K	5-20%	< 1 cm
	Thermal coefficient of expansion; HTGR Graphite - 5%	2%	NA ^c
	Density (estimated from changes in length, diameter, porosity, etc.) HTGR Graphite - 0.5%	0.2%	NA
Material mechanical/electrical properties	Irradiation creep HTGR Graphite - 3-4%	NA	NA
	Young's modulus HTGR Graphite - 3-4%	NA	NA
	Electrical resistivity HTGR Graphite - 3-4%	NA	NA
	Poisson's ratio HTGR Graphite - 3-4%	NA	NA
	Fracture toughness, shear strength HTGR Graphite - 3-4%	NA	NA
Material irradiation neutron flux for estimating fluence	Thermal neutron flux - $\sim 1.5 \times 10^{14}$ n/cm ² -s	1-10%	5 cm (axially)
	Fast neutron flux (E > 1 MeV) - $\sim 1.5 \times 10^{14}$ n/cm ² -s	15%	5 cm (axially)

a. Representative peak values, accuracy, and resolution are based on engineering judgement and are preliminary.

b. Only LWRS and NGNP irradiation information available for a limited number of parameters at this time.

c. NA-Not Available.

2. BACKGROUND

This section describes commonly used flux detection sensors in typical MTR irradiations. A brief overview discussing their theory of operation, basic construction, deployment considerations, data collection considerations, and basic advantages/disadvantages is given to emphasize advantages of the HT MPFDs and how they uniquely address irradiation testing needs for DOE-NE programs.

2.1. Flux Sensors

Accurate measurement of the time-dependent neutron field, the neutron flux, and the integral exposure or the neutron fluence that a sample receives is needed in order to assess effects of neutron interactions on the samples under investigation. Some tests only require that the overall neutron fluence be known. Hence, passive methods are used to measure the neutron irradiation after the sample has been removed from the reactor. Many DOE-NE irradiation tests rely on passive neutron fluence measurement techniques. However, as discussed in Section 1, many DOE-NE programs have identified candidate new materials and fuels whose performance evaluations require high accuracy/high resolution real-time fast and thermal flux measurements. Such measurements are typically accomplished with Self-Powered Neutron Detectors (SPNDs) or fission chambers. As indicated in Table 3, there are advantages and disadvantages with each type of detector.^{6 through 8} This section describes typical examples of each type of sensor along with their limitations in irradiation tests to emphasize the need for this project to develop HT MPFDs for use in DOE-NE irradiation test programs in higher flux US MTRs, such as the 250 MWt Advanced Test Reactor (ATR) and the 100 MWt High Flux Isotope Reactor (HFIR) and the Transient REactor Test (TREAT) reactor.

Table 3. Summary of advantages and disadvantages of typical flux sensors

Flux Sensor	General description	Typical characteristics	
		Advantages	Disadvantages
Flux Wires and Foils	Utilizes neutron activation of materials to emit measurable gamma-rays	Small Thermal/Fast neutron response No power source Simple/robust Easily installed	Not real time Rigorous post-analysis Integral measurement
Self Powered Neutron Detectors	Generates measurable electrical current under neutron irradiation	Real time Small Thermal neutron response No power source Simple/robust	Need calibration No fast neutron response Low sensitivity Delayed response Need electrometer
Fission Chambers	Fissionable material interactions provide measurable electrical pulses	Real time Small Thermal/fast neutron response Variable sensitivity	Need power source Fragile
Micro-Pocket Fission Detectors	Fissionable material interactions provide measurable electrical pulses	Real time Small Simple/robust Simultaneous thermal/fast neutron response with temperature measurement Variable sensitivity Low power requirements	Need power source

2.1.1. Flux Wires and Foils

Flux wires and foils are the most commonly used passive methods for local neutron activation spectrometry. A flux wire or foil is simply a material of known composition and purity that is placed in a neutron field. When placed in a neutron field, different elements (and different isotopes of the same element) interact differently with respect to the energy of the incident neutron. When the interaction product is radioactive, the resulting induced activity can be measured and correlated to the integral incident neutron exposure.⁶ This measurement and correlation can only be performed after the irradiation is complete and the flux wire or foil is removed from the reactor.

Deployment simply requires that the flux wires or foils be securely placed in the reactor in appropriate fixturing (Figure 1). Hence, they can be placed at nearly any location in the reactor. Some materials can be placed directly in contact with the reactor primary coolant, whereas other more fragile materials must be encased in low cross-section materials that don't affect their neutron exposure. Data collection requires removing the radioactive flux wires or foils from the reactor and placing them on a gamma ray detector and counting them for a specific amount of time. The counting information and material information, along with the irradiation time and the time since the end of the irradiation, can then be used to calculate the neutron flux. The advantages of this method are: very small sensors can be used; it is generally very accurate; it can be used for high and low flux levels; and it gives information about the incident neutron energy. The disadvantages are that it is not real-time and that it requires handling radioactive material.

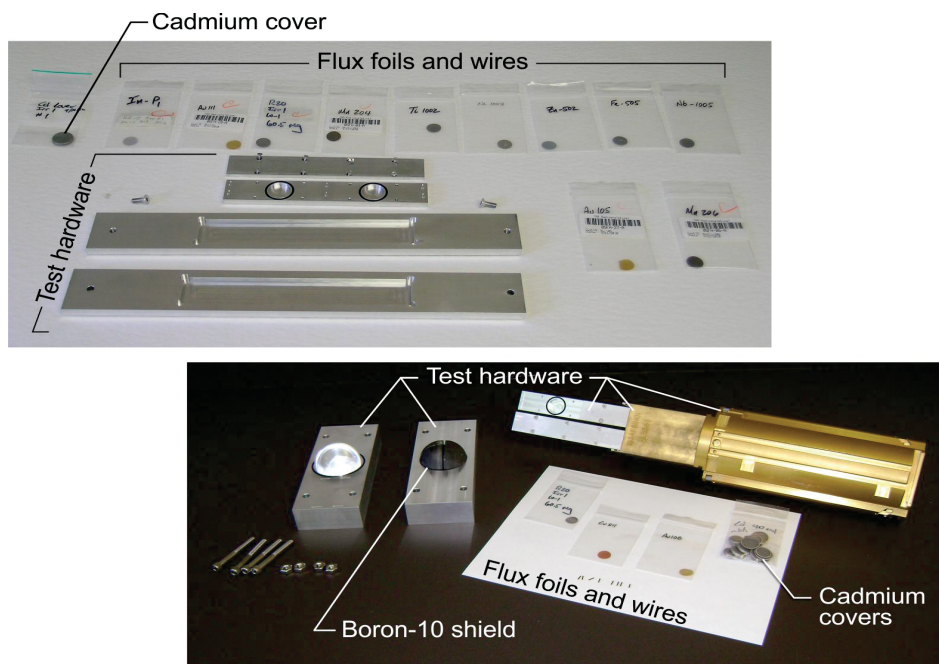


Figure 1. Representative flux wires, flux foils and test hardware used at INL

2.1.2. Self Powered Neutron Detectors

SPNDs are a commonly used real-time method for monitoring local neutron flux. SPNDs are built using materials that become radioactive in a neutron field and produce a small current which is correlated to the neutron flux (Figure 2). SPNDs are built around a central electrode, known as an emitter, which is composed of a metal with a relatively high neutron capture cross section.⁶ Typical emitters and their properties are listed in Table 4. The central electrode is surrounded by an electrical insulator and an outer electrode, known as a collector. The outer electrode serves as a boundary between the reactor coolant and the emitter and insulator. The current between the emitter and collector is measured via external circuitry. It can take time for the current to build up in the SPND; hence, there is a delay time from several seconds to a few minutes before the signal is generated. In addition, if the neutron signal is suddenly changed it will take several minutes for the signal to stabilize.

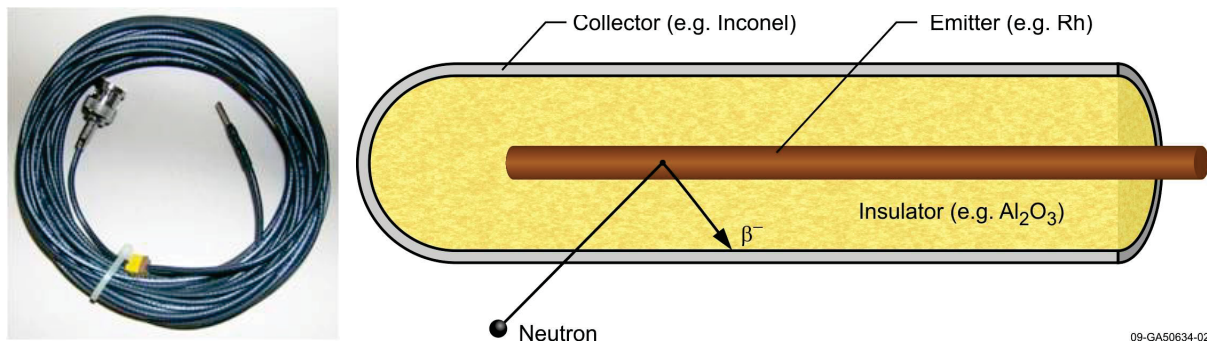


Figure 2. Representative Self-Powered Neutron Detector (SPND) and component sketch

Table 4. Typical emitters for SPNDs

SPND Emitter	Sensitivity (A/m)/[2x10 ¹⁷ neutrons/(m ² s)]	Response
Rhodium	2.4 x 10 ⁻⁶	Delayed
Vanadium	1.5 x 10 ⁻⁷	Delayed
Cobalt	3.4 x 10 ⁻⁸	Prompt
Molybdenum	1.7 x 10 ⁻⁸	Prompt
Platinum	2.6 x 10 ⁻⁷	Prompt

Deployment requires a space large enough to fit the SPND, typically 1.5-6 mm in diameter, and the leads to be run from the experiment location to outside the reactor for signal processing; hence, SPNDs can only be deployed in specific experiment locations in the reactor. Data collection requires the SPNDs to be connected to sensitive low current measurement equipment. The SPNDs must be previously calibrated in a known neutron field so that the current response can be correlated to the incident neutron flux. The advantages of this method include: it is self powered; it provides near real time neutron flux measurement; and it is a robust sensor. The disadvantages include: it has delays associated with its response; it requires a large neutron flux to generate a large current; it requires a calibration; it is generally limited to thermal neutrons; and it requires signal leads from the experiment to outside the reactor.⁶

2.1.3. Fission Chambers

Fission chambers are another commonly used real-time method for monitoring local neutron flux. As shown in Figure 3, fission chambers are built using two electrodes, one has a fissile material deposit that emits fission fragments when placed in a neutron field. These fission fragments produce an instantaneous current pulse between the electrodes that have a polarization voltage applied between them. The resulting pulse is measured via external counting electronics. The space between the electrodes must have a fill gas to aid detector performance. Like SPNDs, the outer electrode serves as a boundary between the reactor coolant and the inner materials. The fissile material deposit can be chosen such that the detector is sensitive to either thermal neutrons or fast neutrons. Fission chambers can be operated in three different modes: pulse mode for low power applications that measure the individual pulses, current mode for midrange power applications where pulses are so frequent that they can't be separated and produce a continuous current, and Campbelling mode for high power applications where the variance of the signal is characterized and correlated to the incident neutron flux.⁶

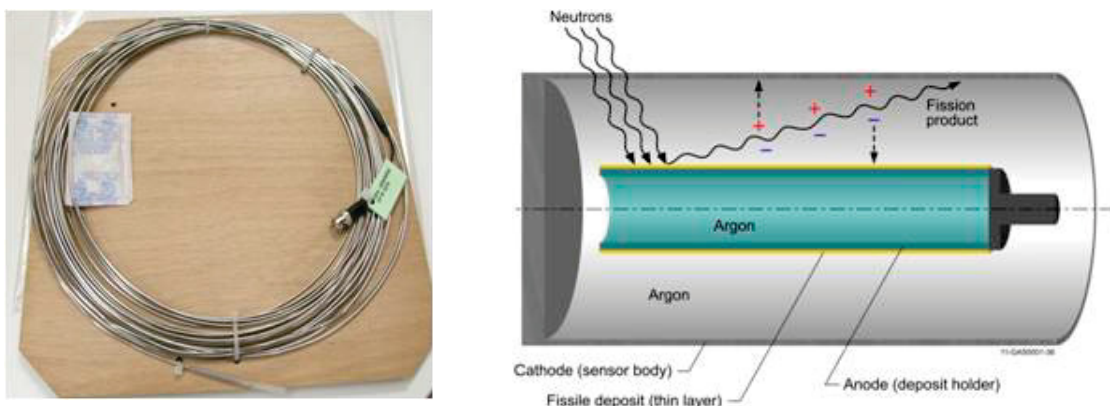


Figure 3. Representative fission chamber and component sketch

Deployment requires a space large enough to fit the fission chambers, typically 1.5-150 mm diameter, and leads from the fission chamber location to outside the reactor vessel for data display and storage. Hence, fission chambers can only be deployed in specific experiment locations in the reactor and sometimes are deployed outside the reactor core for large fission chambers or to improve fission chamber lifetime. Fission chamber response is correlated to incident neutron flux by accurate measurement of the fissionable deposit mass or by prior calibration in a neutron field. The advantages of this method include: it is real-time; it can be used for small and large neutron fluxes; it can be designed for thermal and fast neutron flux measurements; and it is more accurate than SPNDs.⁶ The disadvantages include: it is more delicate; it can require a larger space in the reactor; and it requires signal leads from the experiment to outside the reactor.

2.1.3.1. Miniature and Sub-Miniature Fission Chambers

Irradiations at US MTRs are typically performed at high fluxes using small samples with limited space for instrumentation. In addition, some DOE-NE irradiations must be performed at high temperatures (up to 2600 °C). Fission chambers must be sufficiently robust to survive these harsh conditions for the length of

the experiment. As such, special fission chambers have been developed or are under development for use in US MTR experiments. The following section discusses several of these in-core fission chambers deemed most promising.

The Commissariat à l'Énergie Atomique et aux Energies Alternatives (CEA) has over 40 years of experience in design, construction and use of miniature and sub-miniature fission chambers for MTR experiments (Figure 4).^{7,8} CEA-developed fission chambers are as small as 1.5 mm diameter for sub-miniature fission chambers and 3 mm diameter for miniature fission chambers. The designs utilize two coaxial cylindrical electrodes of which the center electrode is covered with fissile material. The area between the electrodes is filled with argon gas between 1-10 atm pressure. The complexity of the coaxial design along with the potential for the high pressure fill gas to leak makes these miniature fission chambers more susceptible to damage during harsh irradiations.^{7,8} The CEA fission chamber sensitivity to thermal and fast neutrons is increased by using different fissile material deposits. In addition, as part of the Joint Instrumentation Laboratory, CEA and Studiecentrum voor Kernenergie Centre d'Étude de l'Énergie Nucléaire (SCK•CEN) have also developed a Fast Neutron Detector System (FNDS) for monitoring fast neutron flux in large flux environments. Prior to the development of a new type of fission chamber, CEA models the expected response with specially developed software.^{7,8}

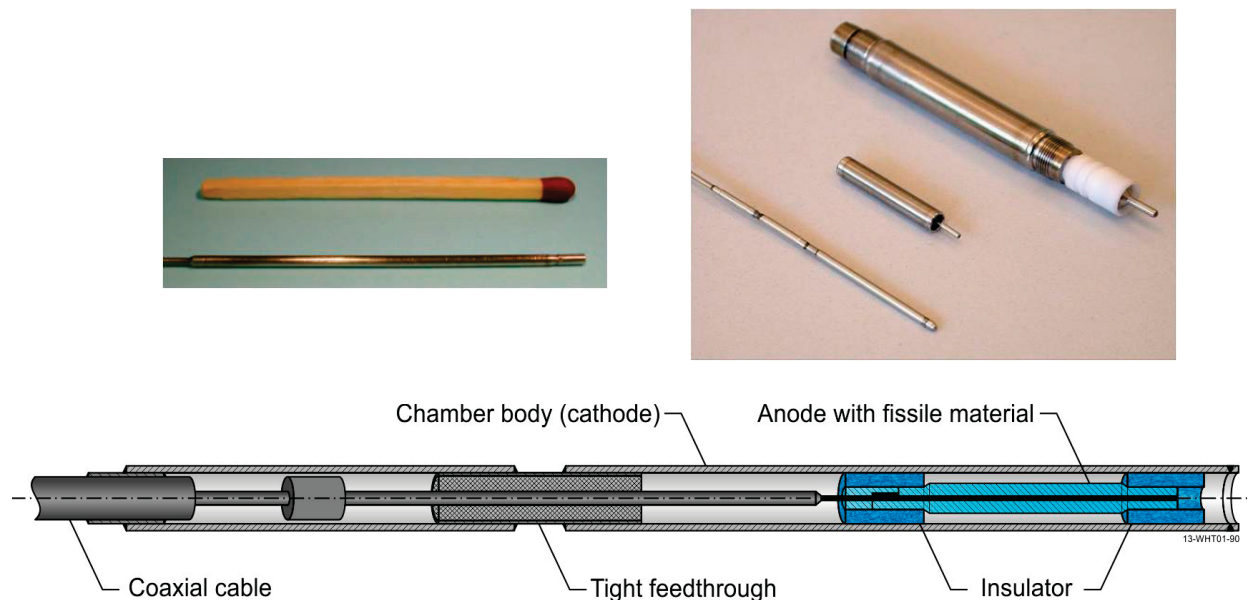


Figure 4. Representative CEA miniature fission chambers and component sketch

2.1.3.2. Micro-Pocket Fission Detector Technology

Within the last decade, efforts were initiated by the Kansas State University (KSU) to develop the MPFD technology that has the potential to simultaneously detect thermal and fast flux along with temperature in a single miniature sensor.^{9 through 14} Initial 'proof-of-concept' evaluations to demonstrate MPFD technology performance were completed at the KSU TRIGA reactor. However, prior to deployment of this new MPFD technology in a US MTR, a more robust sensor design was required. As discussed in Section 3, previous

research conducted through NEET ASI¹⁻⁴ has shown that the MPFD technology could be made robust and was successfully tested in a reactor core.

MPFD technology utilizes the same concept as coaxial fission chambers, but with a different geometry that uses parallel plate electrodes instead of coaxial cylinders. This design is known as a parallel plate fission chamber. However, the MPFD design is distinguished from other fission chambers because their signal is not based on the full energy deposition in the electrode gap from the fission products. This departure from conventional fission chamber design and operating characteristics allows the MPFD technology to have a much smaller chamber size with a much lower fill gas pressure. The MPFD design has excellent discrimination characteristics because the energy deposited by the fission products is much greater than other types of background radiation interactions in the detector. Another benefit is that the small size allows them to have a faster response time. Thus, the smaller MPFD design has the potential to achieve higher count rates than conventional fission chamber designs. The construction materials chosen for the MPFD technology includes temperature and radiation resistant ceramics. All of these characteristics make the MPFD technology well-suited to survive the harsh conditions present in DOE-NE MTR experiments.^{9 through 14}

3. MPFD INVESTIGATIONS

This section presents the scope for this NEET ASI project and summarizes investigation activities completed in this effort. A brief review of collaborator facilities and prior NEET ASI¹⁻⁴ activities to develop the MPFD technology is also presented. In addition, a discussion of additional irradiation opportunities funded by other DOE-NE programs is included.

3.1. Overall Plan and Schedule

This joint INL/KSU/CEA project consists of seven tasks to be completed within three years. Figure 5 illustrates activities to be performed each year of this project and the organizations participating in each activity. Task 1 of this project is devoted to enhancing the previously developed MPFD design¹⁻³ for higher temperature applications and procuring candidate materials. Task 2 activities included refining construction techniques to fabricate a HT MPFD. Task 3 will evaluate the HT MPFD in high temperature furnaces and autoclaves as well as develop computer based analysis models of the detector performance. Task 4 will focus on evaluations in available irradiation facilities. In Task 5, investigators will procure new materials and refine the HT MPFD design and construction methods as well as update analysis models as needed depending on Task 4 results. Task 6 will repeat Task 4 evaluations and compare and contrast design changes in actual and analytical evaluations. Task 7 includes all of the ongoing reporting and project management to manage subcontracts and prepare annual reports and publications.

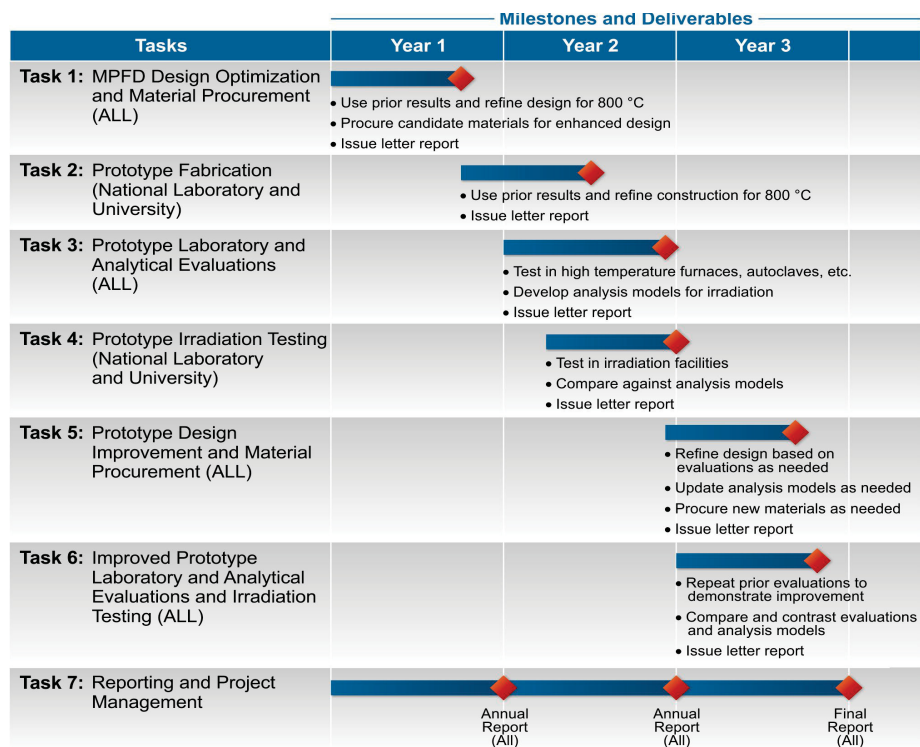


Figure 5. MPFD development tasks

3.2. Prior Activities and Capabilities

Research activities for previous development of the MPFD technology is discussed in the following sections. In addition, the capabilities of the collaborating organizations are summarized to provide a brief overview as why the research team was chosen.

3.2.1. Prior KSU MPFD Design and Evaluation Activities

Initial development of prototype KSU MPFD designs began in the early 2000's at KSU using large (~25 mm x 25 mm) alumina substrates with deposited electrical contacts (Figure 6).^{9 through 14} Evaluations for depositing neutron reactive materials explored various methods, including boron with physical vapor deposition, uranyl nitrate solution with an eyedropper, and uranium and thorium coatings using electrolysis. After the neutron reactive material was applied, the detector substrates were bonded together using epoxy in an argon atmosphere. Initial KSU MPFD prototypes were tested in a neutron beam at the KSU TRIGA research reactor with successful results. However, it was recognized that the manufacturing process was not ideal to produce detectors for in-core applications

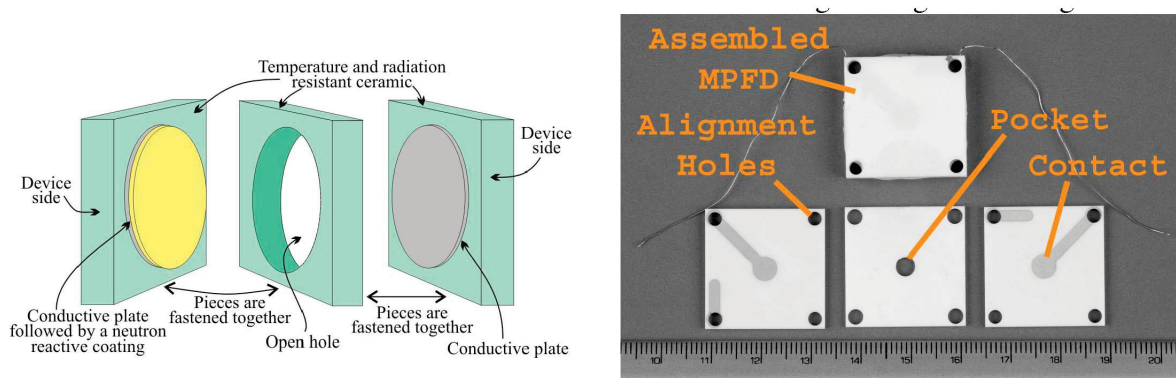


Figure 6. KSU MPFD component sketch (left) and initial KSU prototype MPFD (right)

The following years lead to further refinements of the KSU MPFD design that combined three chambers (one fast, one thermal, one background) and provided space for a thermocouple on a single substrate. Although the three chamber design incorporated multiple sensors in a single package, the new design was smaller (~25 mm x 6 mm) than previous designs.^{9 through 14} Electroplating proved to be the most reliable method to deposit the neutron reactive materials. Uranium was used for thermal neutron detection, and thorium was used for fast neutron detection. The background chamber was added to demonstrate that the neutron sensitive chambers were insensitive to background radiation. The provision for a temperature measurement with a Type K thermocouple wire bonded to the detector substrate was included; however, the attachment method was not ideal for optimal thermocouple performance.

Deployment of KSU MPFDs in their TRIGA reactor had limited success.^{9 through 14} The testing resulted in the largest amount of the MPFD technology ever installed in a reactor core, 225 detectors. However, only about one third of them functioned; and the detectors that did function had issues with cross-talk between the output signals. The operational issues were due to water flooding into the probe tubes, wiring isolation

issues, and improperly selected electronics. The probe tubes were constructed using three different diameters of aluminum tubes welded together using a TIG welder. The tubes were checked for leak tightness in a shallow water tank, but several tubes leaked when deployed in the KSU TRIGA reactor tank.

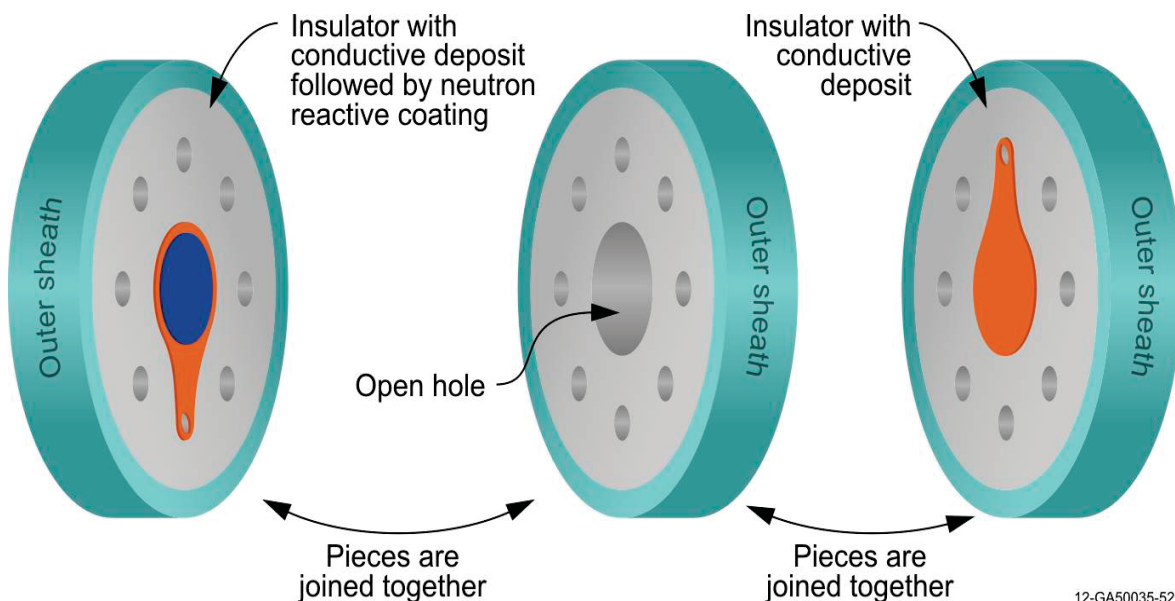
The wiring was required to exit the KSU designed MPFD perpendicular to the substrate and immediately make a 90 degree bend to travel inside a protective metal sheath. The wires were separated from each other using wire guides, but were not shielded from each other for their entire length. After deployment in the reactor, it was discovered that several of the wires were in contact with each other and the outer sheath, rendering many detectors unusable.

In addition, the KSU designed MPFDs utilized a single power source to reduce the amount of wires extending the length down to the reactor core. After deployment, it was determined that this arrangement introduced cross-talk issues that led to multiple pulses from a single detector that shared a power supply with other detectors.^{9 through 14}

3.2.2. Prior NEET ASI MPFD Design and Evaluation Activities

The MPFD technology gained interest for further development during a 3 year project to refine previous MPFD designs for use in MTR irradiations. A NEET ASI project, *Micro-Pocket Fission Detector*, was awarded in 2012 to develop a MPFD design suitable for use in these harsh irradiation conditions.¹⁻³

This MPFD design for MTR irradiations uses a round stackable geometry (Figure 7). The round design characteristic solves three problems associated with the previous KSU design. First, a round geometry is more suitable for installation in leak-tight swaged, drawn, or loose assembly tubes, eliminating water ingress problems. Second, the round tube geometry includes insulation for the wires along their entire length from the reactor core to the data acquisition system, eliminating the potential for wire-to-wire contact. Third, the wire connections are made through the detector substrate, eliminating a 90 degree wire bend used in the original KSU MPFD design (Figure 8).



12-GA50035-52a

Figure 7. Round geometry MPFD design suitable for MTR irradiations

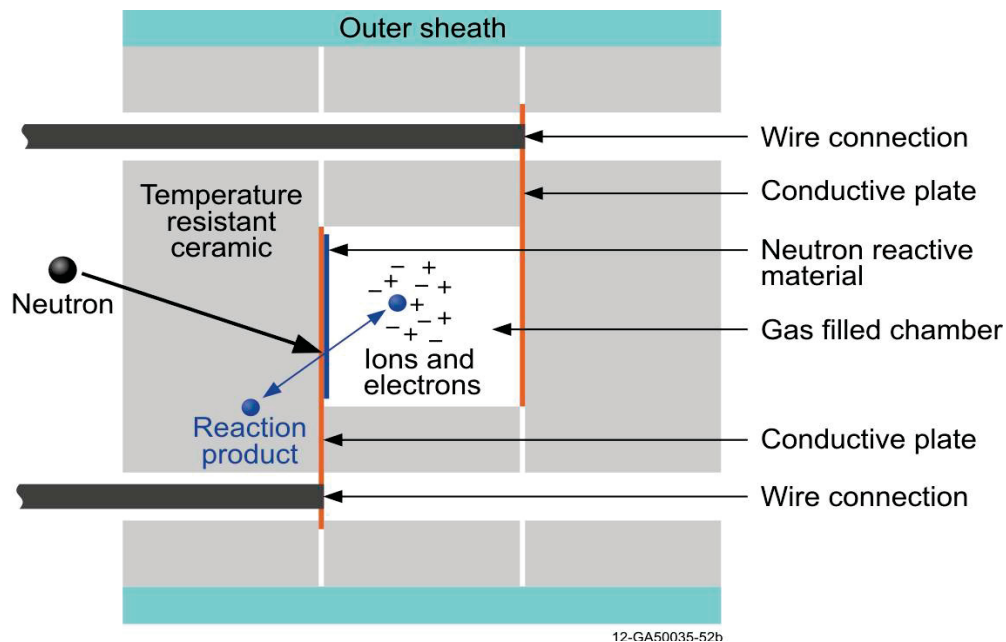
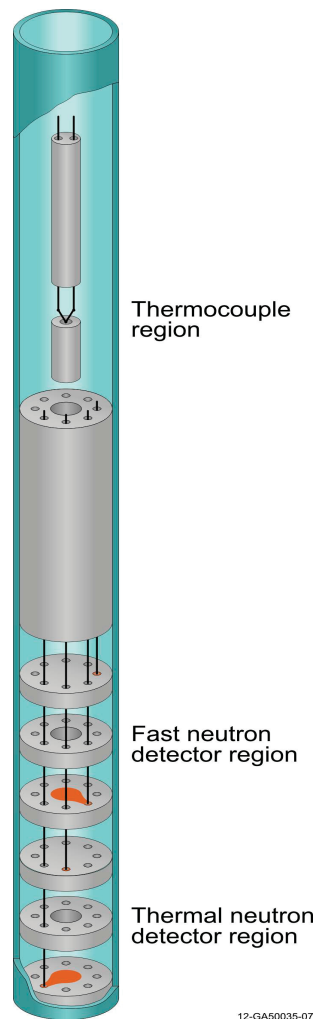


Figure 8. Component diagram of MPFD design showing wire locations

It was also determined that the third chamber used for detection of background radiation was unnecessary because prior KSU testing demonstrated the MPFD design is background insensitive due to the small size. An additional benefit of the stackable design is the potential for multiple detectors of varying sensitivities to be added as customer requirements dictate. The thermocouple is placed directly above the fission chambers in the loose assembly (Figure 9). This eliminates the additional wire bonding required in the KSU design, thus improving thermocouple performance and robustness. In addition, an INL invention disclosure record was filled on the MPFD technology.¹⁵

While undergoing construction, the fabrication methods were assessed by considering thermocouple response, x-ray observations, helium leak checks, and resistance and continuity measurements. The material deposition was carefully monitored to produce robust electrical contacts and fissile material deposits. Sheath and weld integrity was evaluated using helium leak testing equipment to verify that the assembly meets requirements for the ASTM leak rate of mineral insulated thermocouples because that was the most analogous test.¹⁶ Leak testing revealed that the round geometry MPFD leak rates were well below the acceptable ASTM leak rate limits.

After an acceptable leak rate was verified, external wiring was added to allow the round-geometry MPFDs to be connected to a data acquisition system (Figure 11) for long duration tests in a high temperature furnace. These evaluations were performed in furnaces to verify the internal wiring connections and thermocouple performance would survive at 500 °C for 1000 continuous hours. The evaluations indicated the wiring connections and thermocouple performance could survive at these elevated temperature for more than 1000 hours. As shown in Figure 12, the thermocouple inside this design of MPFD continued to provide a temperature reading consistent with the Type K reference thermocouple included in this test. Likewise, connections to flux detector components survived as indicated by their continuous and consistent voltage signal throughout the test.



12-GA50035-07

Figure 9. Initial MPFD design for MTR irradiations



Figure 10. Leak testing an enhanced MPFD in specialized HTTL leak detection system

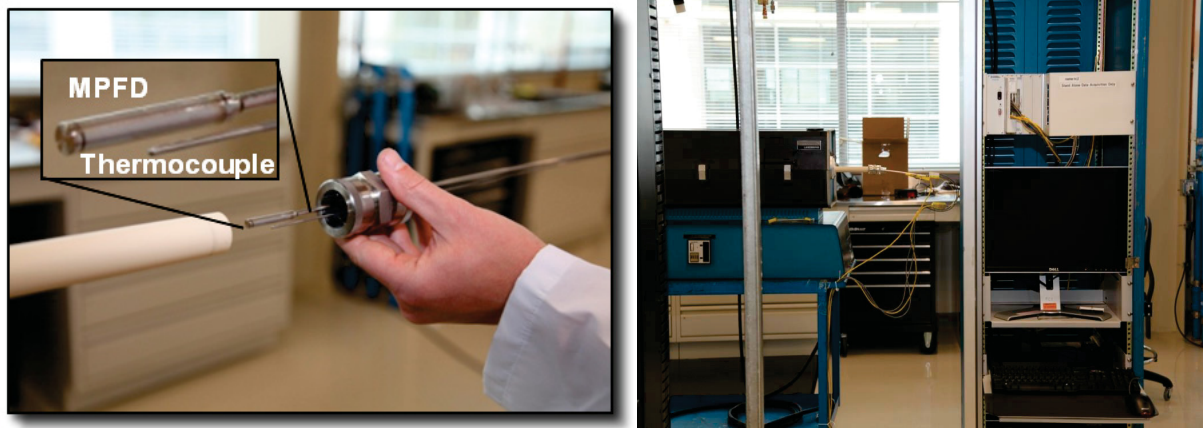


Figure 11. Round geometry MPFD installation into furnace and data acquisition system

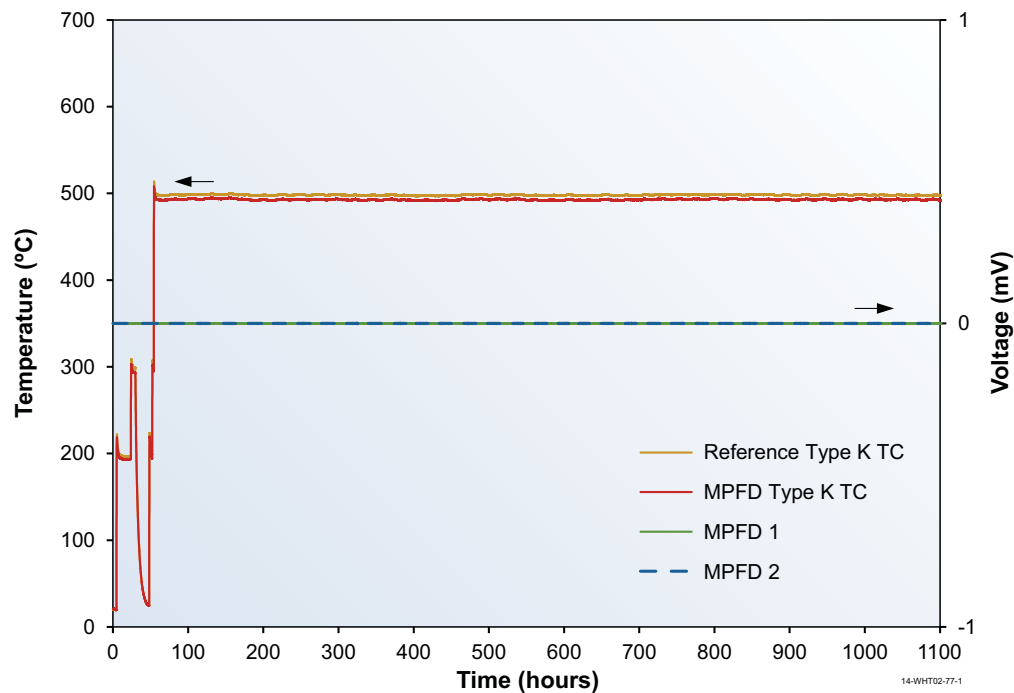


Figure 12. Round geometry MPFD long duration high temperature test results

Radiation sensitivity testing was performed at the KSU TRIGA reactor by installing the round geometry MPFDs in the central thimble of the reactor core (see Figure 13) using KSU's specially developed signal amplifiers. The round geometry MPFDs were tested in-core at thermal and fast fluxes of $2.00 \times 10^{13} \text{ n/cm}^2\text{-s}$ and $2.00 \times 10^{14} \text{ n/cm}^2\text{-s}$, respectively. Typical detector pulses are shown in Figure 14.

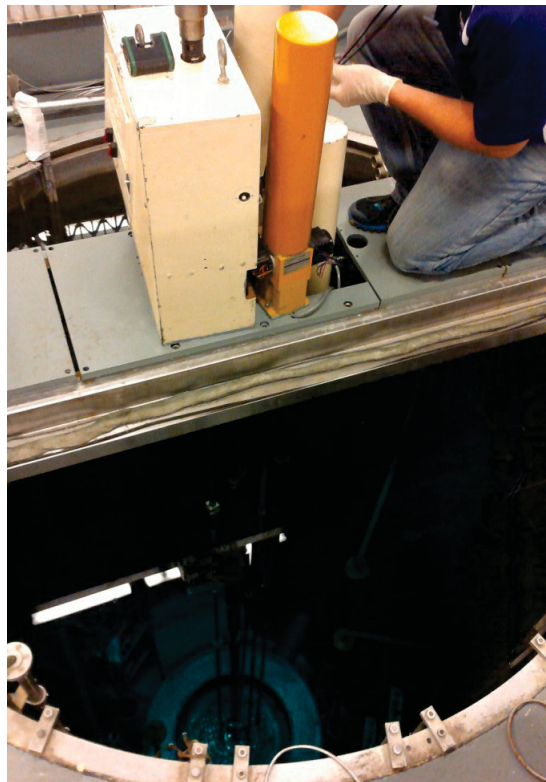


Figure 13. MPFD installation in the central thimble at KSU TRIGA reactor

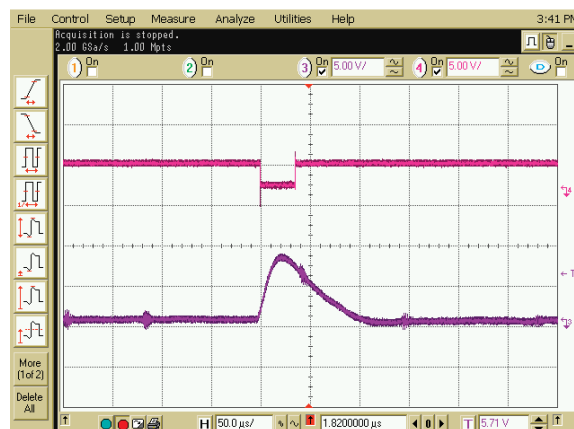


Figure 14. MPFD pulses during KSU TRIGA reactor testing

3.2.3. KSU Research Facilities

KSU experience in producing the initial MPFD prototypes was beneficial for producing updated versions of neutron detectors that utilize the MPFD technology. KSU has several specialized laboratories to aid in the construction and testing of various radiation detectors.¹⁷ through ¹⁹ KSU specialties in the fabrication and implementation of the MPFD technologies include: building, testing, and optimizing detector ampli-

fier boards for the MPFD technology; innovative fissile deposition for the MPFD technology; and evaluating the detectors in their nuclear reactor. This section summarizes KSU capabilities utilized for this effort.

3.2.3.1. SMART Laboratory

The Semiconductor Materials and Radiological Technologies (SMART) Laboratory at KSU is a unique facility dedicated to the research and development of new and innovative radiation detector technologies. The SMART Laboratory houses specialized equipment to support that work, including clean rooms, a fission chamber plating station, a linear drive diamond cutting wheel, a diamond wire saw, precision lapping and polishing machines, a 6-pocket e-beam evaporator (Figure 15), an ion mill, a vacuum annealing chamber, microscopes, ovens, grinders, an assortment of furnaces for annealing, sintering, diffusions, and oxidations, a scanning electron microscope, an Auger electron analyzing system, IV and CV tracers, radiation sources, and NIM electronics to test and characterize radiation detectors and materials. In addition, the SMART Laboratory completed construction of a new 1000 square foot class 100 clean room facility that has been fully operational as of March 2012.¹⁷

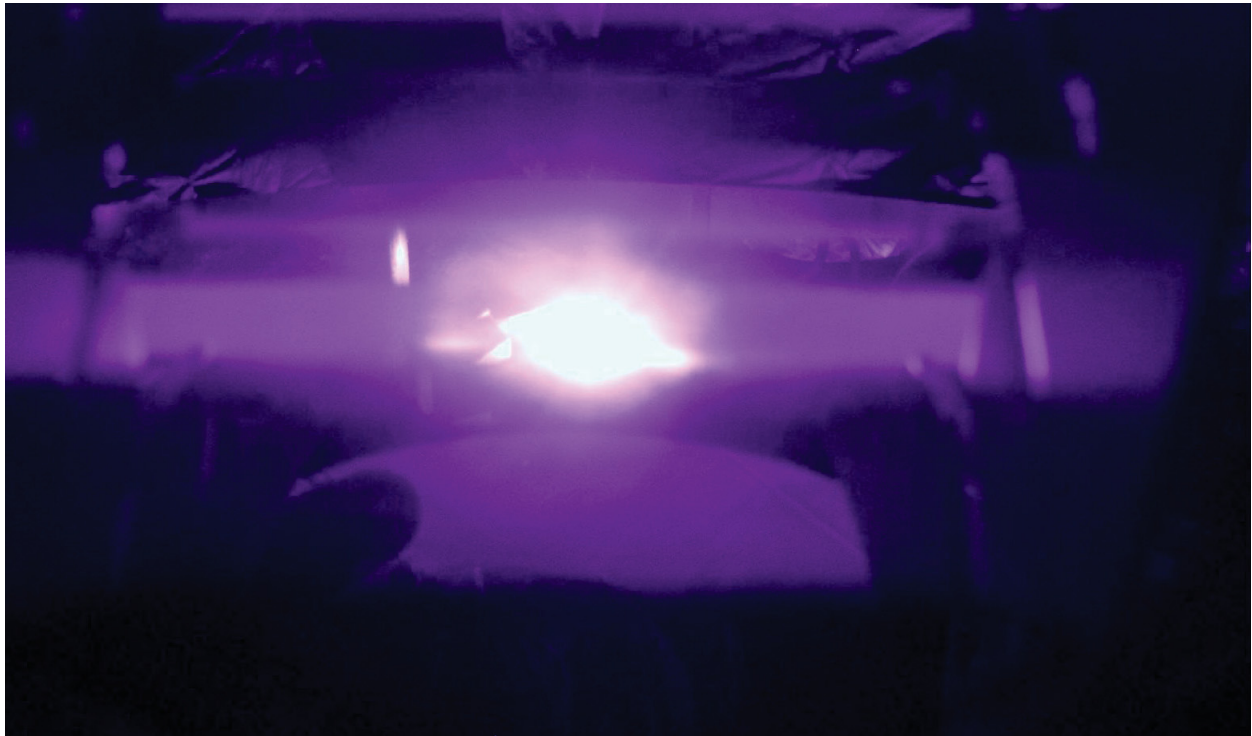


Figure 15. Electron-beam evaporator depositing material at the KSU SMART Laboratory

The facility is available to students and faculty for research and development of sensors. The SMART Laboratory serves as a center for undergraduate and graduate student education as well as a facility to accommodate funded research projects from various government and industrial sponsors.¹⁷



Figure 16. MPFD sample undergoing micro-electroplating of fissile material at the KSU SMART Laboratory

3.2.3.2. TRIGA Mark II Research Reactor

The TRIGA Mark II research reactor at KSU supports education, research, training, and regional industries. The reactor is used extensively by the SMART Laboratory for testing of radiation sensors and irradiation of materials. The KSU TRIGA is licenced to a 1.25 MW steady state operating power and has pulsing capabilities up to ~ 2 GW, making it ideally suited for testing sensors in high-power transients. The peak steady-state thermal flux and fast flux available in-core are approximately 2.50×10^{13} n/cm²-s and 2.50×10^{14} n/cm²-s, respectively.¹⁸ Sensors can be tested from source range to full power in reactor beam ports or directly in the reactor core. In 2015, the Intra Reflector Irradiation System (IRIS) was installed in the reactor to aid in irradiations. IRIS is a system of in-core dry tubes that allows in-core sensor evaluations to be performed without a concern of water ingress as shown in Figure 17. The ease of access to this reactor facility makes it an asset to the SMART Laboratory for in-core neutron detector testing.¹⁸

3.2.3.3. Electronics Design Laboratory

The KSU Electronics Design Laboratory (EDL) works closely with the SMART Laboratory to develop the required electronics needed to support sensor research and development. EDL's focus is to provide researchers access to advanced electronics and assist with integrating electronics technology into research programs. EDL is an in-house design service for KSU researchers. The staff has both industry and research experience with analog instrumentation and digital design, as well as software development, system engineering, system construction, and project management.¹⁹



Figure 17. KSU Intra Reflector Irradiation System (IRIS) (left) at KSU TRIGA reactor (right)

3.2.4. INL Research Facilities

The INL's role in developing the MPFD technology utilizes its experience in developing and deploying unique in-pile instrumentation suitable for irradiation testing programs that operate in high flux and high temperature conditions. This section summarizes INL capabilities utilized for this effort.

3.2.4.1. High Temperature Test Laboratory

INL has several specialized facilities to construct and test high performance instrumentation. The High Temperature Test Laboratory (HTTL) contains specialized equipment to design advanced in-pile sensors and conduct high-temperature testing. HTTL's trained staff evaluates high-temperature material properties and develops custom high-temperature instrumentation for nuclear and non-nuclear applications, including new methods for measuring temperature, thermal conductivity, localized heating, and sample deformation in MTRs. In addition, the HTTL houses specialized equipment to support such work, including high-temperature tube furnaces, a high-temperature vacuum furnace, swagers, a draw bench, a laser welder, a helium leak detector system, a real time X-ray imaging system, several autoclaves, and various high-temperature material property measurement systems to provide comparison data (see Figure 18). HTTL efforts support DOE-NE programs that require specialized in-pile sensors for fuels and materials irradiations.²⁰

In 2014, the HTTL was relocated to a new state-of-the-art facility at INL, the Energy Innovation Laboratory (EIL) (see Figure 18). The inclusion of a clean room capability and additional new state-of-the-art equipment including a three-dimensional computed tomography system, fiber laser welder, and electron beam welder furthers HTTL's role as a world-class in-pile instrumentation development and testing facility.²¹

INL capabilities also include two unique facilities for testing in neutron and gamma fields, the ATR Critical facility (ATRC) and the Health Physics Instrument Laboratory (HPIL). A brief description of each of these facilities is provided below. In addition, calibration activities performed at the ATRC are presented.

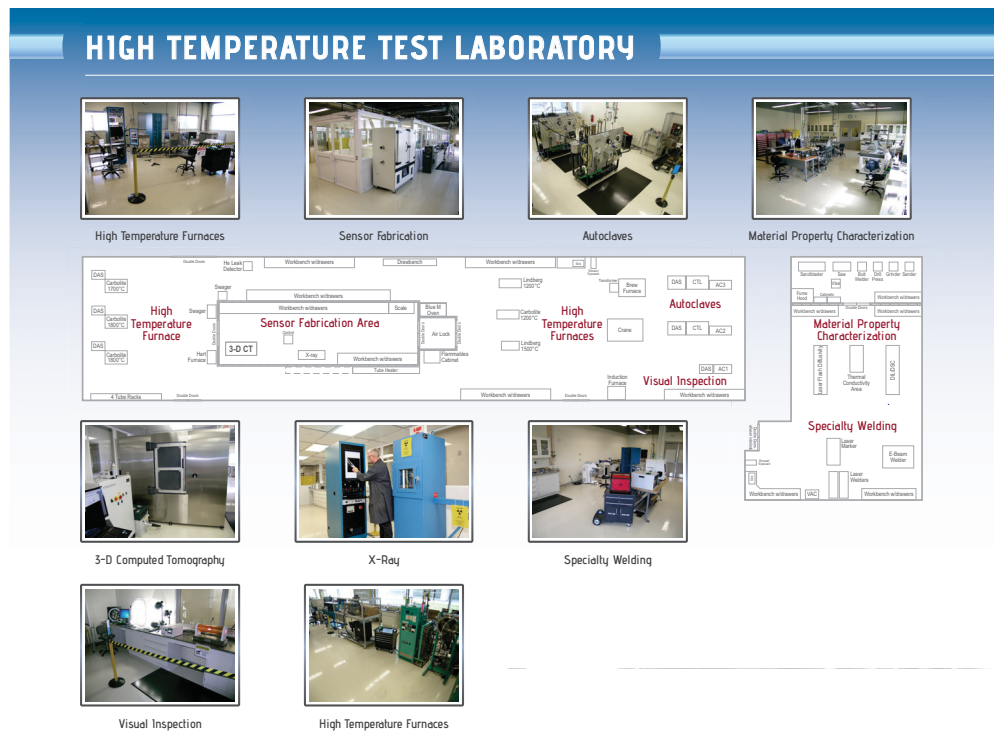


Figure 18. HTTL layout in the new Energy Innovation Laboratory

3.2.4.2. ATRC Facility

The ATRC core is a nearly-identical full-scale nuclear mock-up of the ATR core and can provide valuable insight into the use of advanced sensors prior to deployment in the ATR. The current mission of the ATRC is to obtain accurate and timely data on nuclear characteristics of the ATR core, such as rod worths and calibrations, excess reactivities, neutron flux distribution, gamma-heat generation rates, fuel loading requirements, and effects of the insertion and removal of experiments. The ATRC provides a necessary supplement to analytical reactor methods that support ATR operation. Although the ATRC typically operates at power levels of 600 watts or less, its authorized maximum power level is 5 kW. The core power is maintained at low levels to minimize radiation exposures during manual operations required to unload experiments and fuel. As discussed within this section, an ATR NSUF project has developed and installed Experiment Guide Tubes (EGTs) into ATRC irradiation positions (Figure 19). This recently developed specialized fixturing provides INL a unique capability for comparison evaluations of real-time neutron flux detectors.²²

3.2.4.3. HPIL Facility

The HPIL is the primary radiation protection instrumentation calibration facility at the INL. The HPIL includes two irradiators for detector evaluations. The first irradiator at the Low Scatter Facility houses a National Institute of Standards and Technology (NIST) traceable Cf-252 neutron source that provides a 10 rem/hr neutron dose rate. The second irradiator, the Gamma Beam Irradiator houses NIST traceable Cs-137 and Co-60 gamma sources of 800 R/hr and 400 R/hr dose rates, respectively.²³

These irradiators and the ATRC were not able to be incorporated into prior evaluations for the previous NEET ASI MPFD project due to time constraints associated with evaluations scheduled at the KSU TRIGA reactor. It is planned that the NEET ASI HT MPFD project will use the irradiators and the ATRC to test the neutron sensitivity and the gamma-ray insensitivity of the HT MPFDs.

3.2.4.4. Flux Detector Calibrations at ATRC

As noted in Section 3.2.4.2, ATRC research funded by an ATR NSUF program has resulted in neutron detector evaluation capabilities at INL that are beneficial to HT MPFD development. The project, a joint Idaho State University (ISU)/CEA/INL collaboration, was initiated to investigate the feasibility of using neutron sensors to provide online measurements of the neutron flux and fission reaction rate in the ATRC. The project cross-calibrated various activation detectors (foils and wires), miniature fission chambers, SPNDs and back-to-back (BTB) fission chambers in the ATRC. The SPNDs under evaluation had haf-nium, gadolinium and rhodium emitters for thermal neutron detection. The fission chambers under evaluation had U-235 fissile deposits for thermal neutron detection and U-238 fissile deposits for fast neutron detection. The in-core sensors were inserted into the ATRC using EGTs (Figure 19) and also installed in the northwest in-pile tubes.

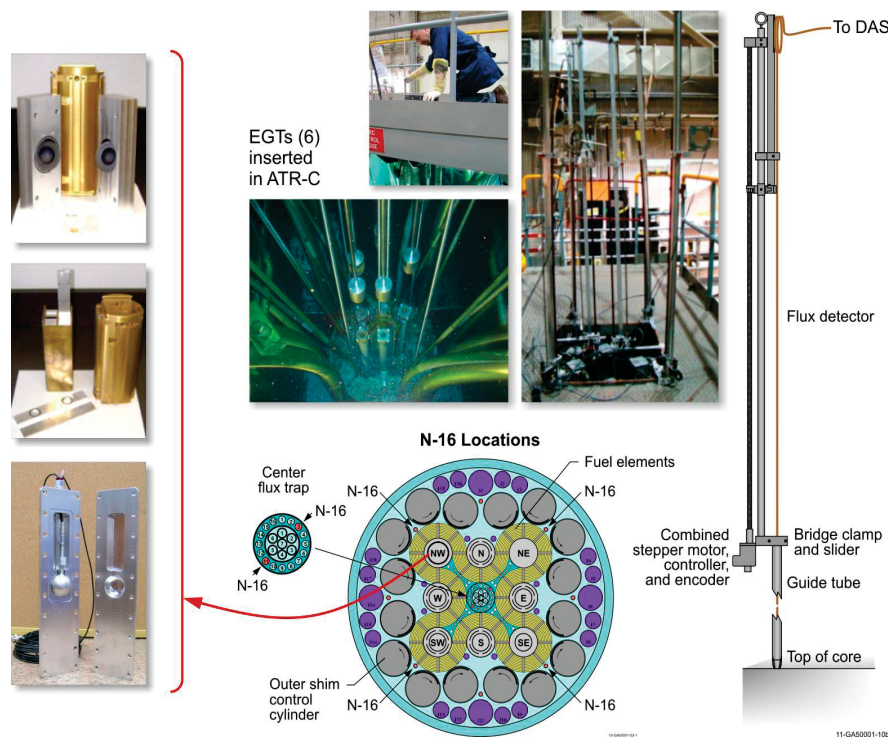


Figure 19. In-core sensors installed in the ATRC

Detector evaluations were performed for a variety of operational conditions in 2013. The reactor was brought to specified power levels between 0.01W and 600W. In addition, measurements were taken with the reactor shut down prior to startup and with the reactor shut down after startup. In order to assess if the detectors could be used to monitor various power splits, the reactor was purposely operated in an unbalanced condition, meaning the reactor power in the NW lobe was purposely changed from the nominal critical power level.

Initially, the flux sensors were verified to be operational; then, the detectors were used to track reactor power. Typical data obtained from axial flux testing and power level testing is shown in Figure 20. In addition to data collection, various insights were gained during the testing of the in-core sensors related to installation, operation, and data analysis. Additional details related to this testing are documented in References 24, 25 and 26.

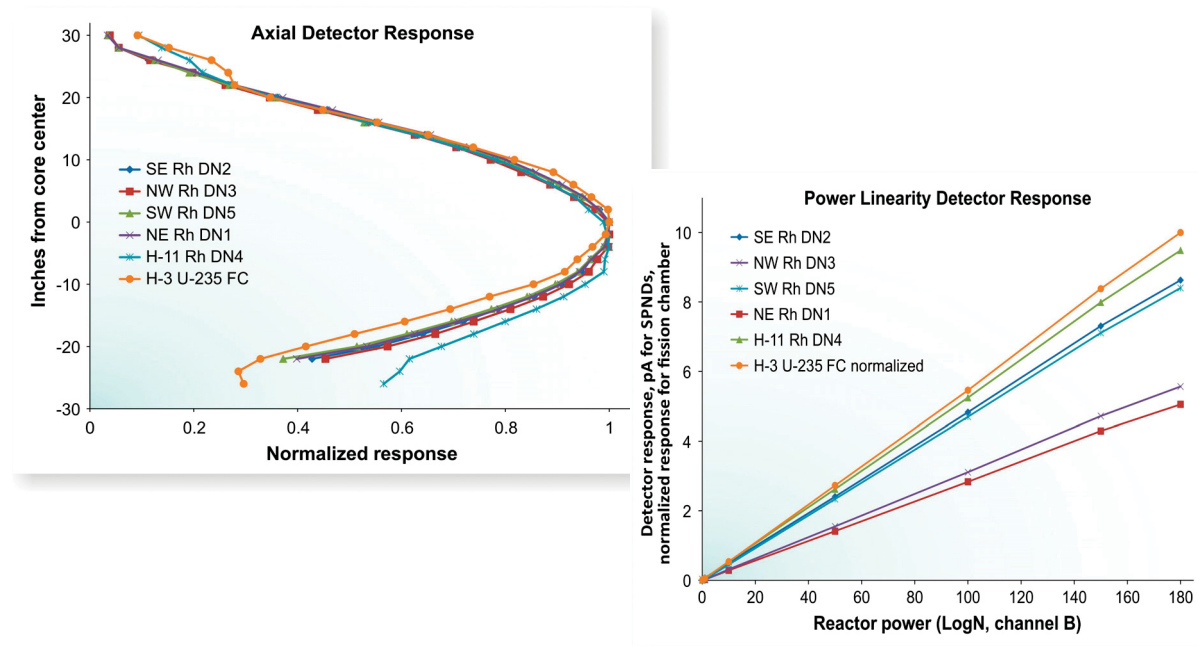


Figure 20. Axial flux profile and power measurements obtained from rhodium SPNDs and a fission chamber

Other cross-calibration fixtures for ATRC were tested in 2014. Specialized fixtures allowed a BTB fission chamber to be inserted into the ATRC. The BTB fission chambers, which are often called 2π fission chambers because they are designed to count almost all fission fragments emitting from a thin deposit in a 2π solid angle, provide the most accurate measure of fission reaction rates and provide accurate cross-calibrations of other detectors such as a HT MPFD. These fixtures place a BTB fission chamber in the ATRC core alongside other in-core sensors to compare their response in near-identical flux conditions. Further experimental details can be found in Reference 26.

3.2.5. CEA Contributions

The CEA has over 40 years of experience in design, construction and use of miniature and sub-miniature fission chambers for MTR experiments. The CEA has developed many specialised computed modeling codes to aid in fission chamber research and development. The CEA contribution to this project was to provide technical guidance and modeling capabilities as needed.

3.3. Current HT MPFD Developments

Previous research conducted through NEET ASI¹⁻⁴ has shown that the MPFD technology could be made robust and was successfully tested in a reactor core. This project continues to further develop the MPFD technology for higher temperature regimes and other reactor applications. This HT MPFD will be developed, fabricated, and evaluated using techniques perfected by INL's HTTL staff during development of other high temperature in-pile sensors, equipment and knowledge possessed by KSU for material depositions and electronics design, and the expertise in miniature fission chamber design possessed by CEA.

3.3.1. FY16 Kickoff Meeting

As part of the FY16 activities, a kickoff meeting was held at KSU on December 17, 2015, (Figure 21) to discuss the HT MPFD design and project schedule, objectives, deliverable and milestones. The meeting demonstrated KSU's progress on electrodeposition of fissile materials. It has been shown that these improvements have resulted in a more consistent fissile material deposition.²⁷

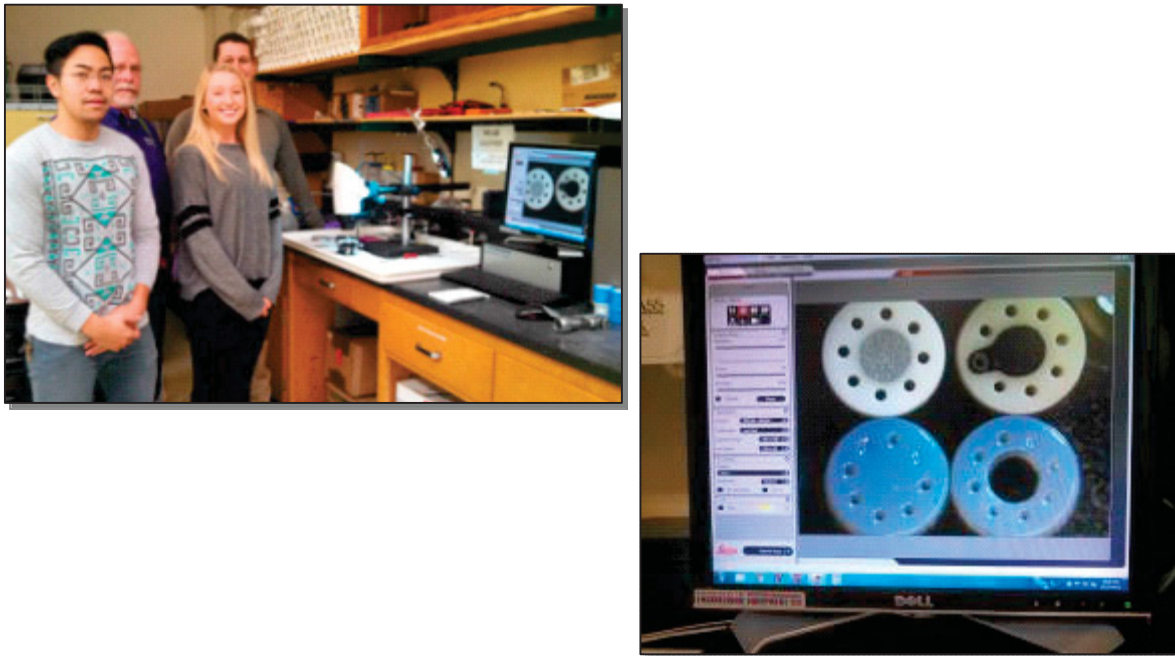


Figure 21. December 2015 project meeting at KSU (left) with close up of KSU fabricated HT MPFD parts (right)

KSU also discussed updating the detector design to move from a parallel plate design to a parallel wire design as shown in Figure 22. This design update allows the MPFD to maintain the electric field in the gas region (Figure 23), but to be constructed without the need for the conductive adhesive as used in previous designs. Removal of the conductive adhesive eliminates the potential failure mechanism of a breakdown on the adhesive in a radiation field. This design change is a significant improvement in HT MPFD survivability in a long duration irradiation test.

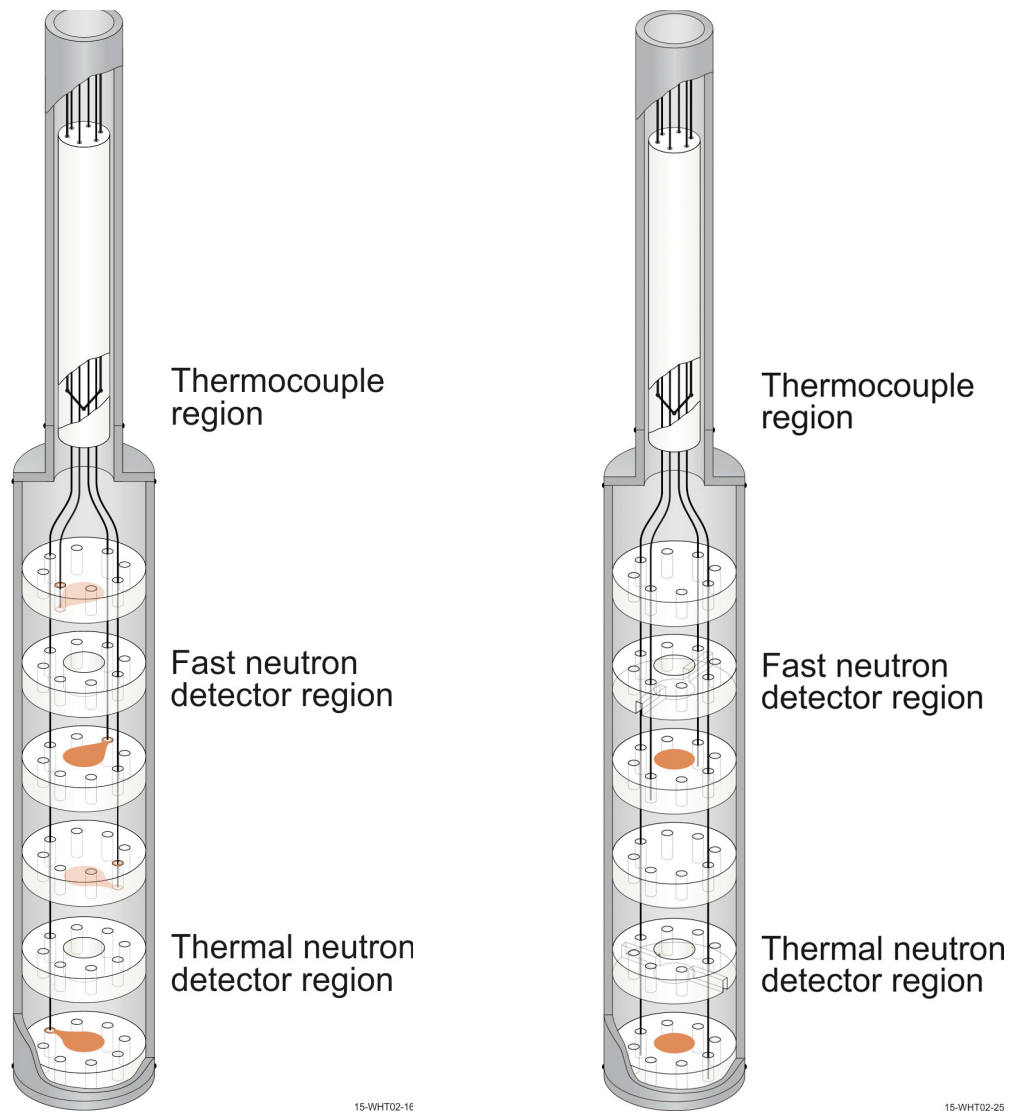


Figure 22. HTMPFD parallel plate design (left) and parallel wire design (right)

In addition, KSU determined that previously developed amplifier electronics would not operate as required for upcoming MPFD deployments. This lead KSU to contract Mesytec for development of a specialized detector readout system as discussed in section 3.3.3. These detector readout modules have been designed and build specifically for the characteristics of HT MPFDs. In addition, other research projects deploying the MPFD technologies were discussed including the recently awarded NEAMS IRP with the University of Wisconsin-Madison.²⁸

3.3.2. HT MPFD Fissile Material Deposition

The KSU SMARTLab is responsible for plating all coatings required for HT MPFD construction. Two plating steps are performed; the first step is to plate the electrical contacts or electrodes on the alumina substrates. The second plating step is to deposit the fissile material on one of the electrical contacts.

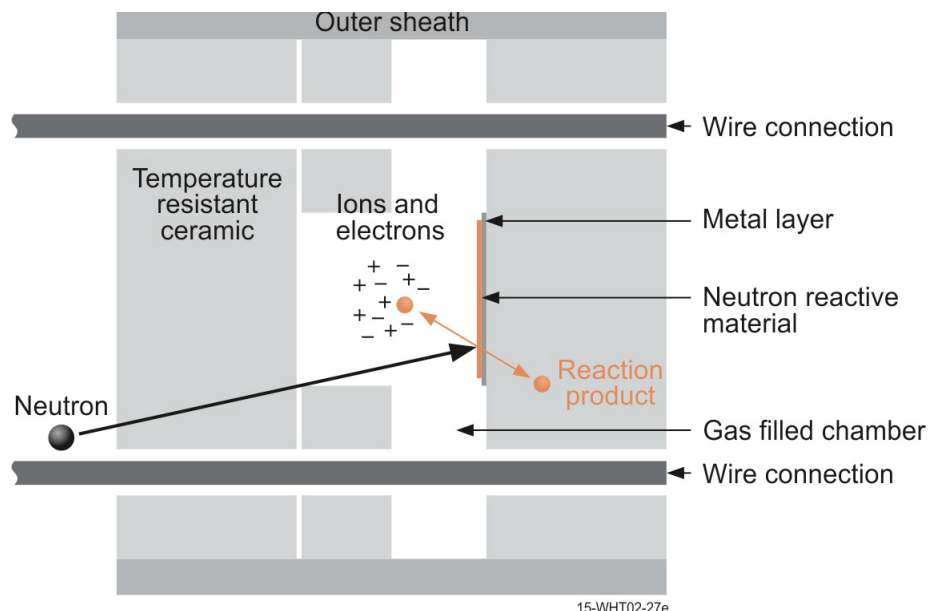


Figure 23. HTMPFD parallel wire diagram

The initial plating to deposit the electrical contacts is performed using an electron-beam evaporator (Figure 15). The plating requires custom shadow masks for the HT MPFDs to be machined using a micro-milling machine at KSU. The purpose of the shadow masks is to selectively deposit the contact material that defines the electrical contacts and wire-bonding pad. Titanium and platinum are used as the contact materials. A thin layer of titanium is used to attached to the alumina, then a thicker layer platinum is used for the electrical contact and to provide a basis for the fissile material electroplating.

Once the electrical contacts are in place, the second plating step will deposit the material. Natural uranium is used for thermal neutron detection and natural thorium is used for fast neutron detection, but other fissile materials could be deposited to alter signal response as requested by irradiation programs. Standard micro-electroplating methods are used to deposit the fissile materials. The small substrates and thin depositions utilize a specialized nA-current electrodeposition system at KSU (Figure 24).

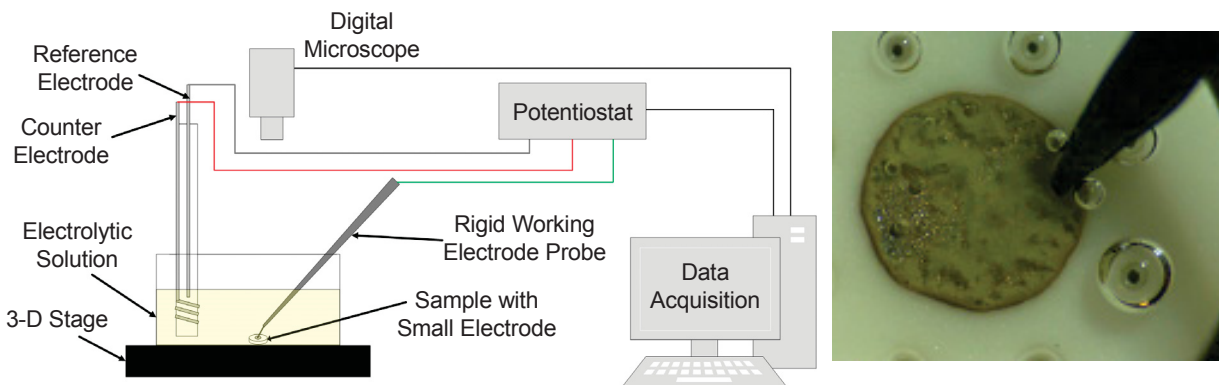


Figure 24. Electrodeposition system at KSU (left) with HT MPFD under deposition (right)

Contact deposition quality has been verified by using an optical profilometer, x-ray fluorescence analyzer (XRF) (Figure 25), scan electron microscope (SEM)²⁷, and electrochemical analyzer. In addition, the fis-

sile deposit characterization is completed by using a specialized 2π solid angle gas detector to verify the amount of material deposited is consistent with predictions.

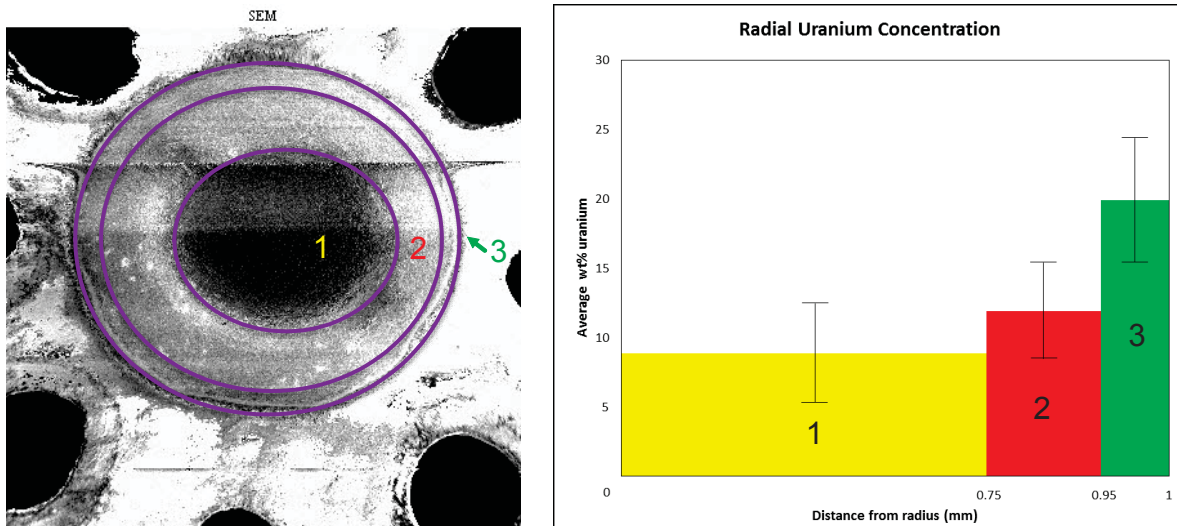


Figure 25. Electrodeposited sample of uranium under analysis with XRF

Addition independent verification of the fission material is under investigation at INL using a newly procured 3D laser scanning confocal microscope. Initial investigations of the MPFD surface finish are shown in Figure 26.

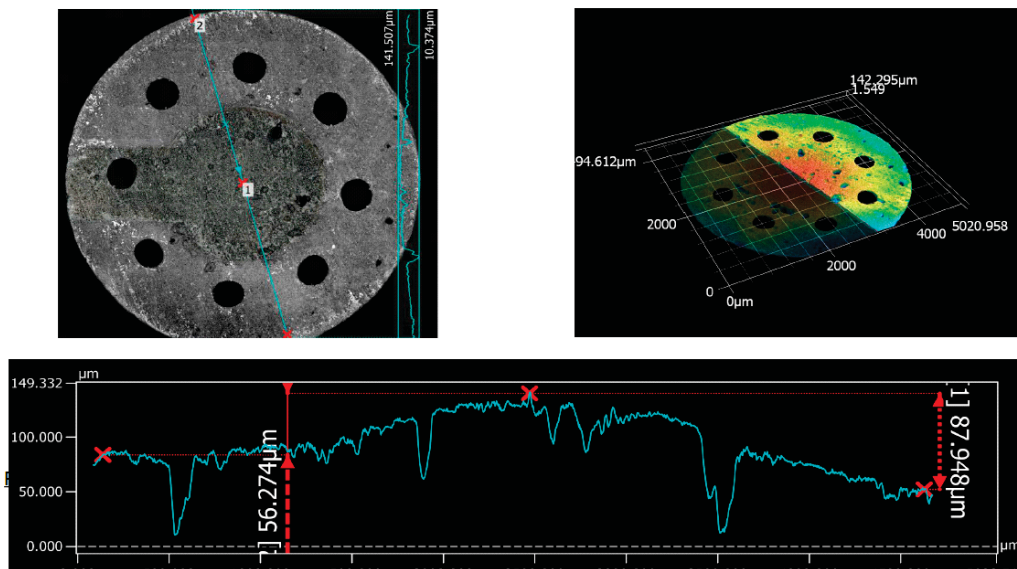


Figure 26. Surface profile of HT MPFD using 3-D laser scanning confocal microscope

The TREAT IRP is also funding independent characterization of KSU's fissile material deposition process used in the fabrication of HT MPFDs. Initial investigations have focused on cross-calibrations against the BTB fission chamber fissile material depositions. These investigations are being performed at the Idaho State University Reactor Laboratory with a neutron source as shown in Figure 27.

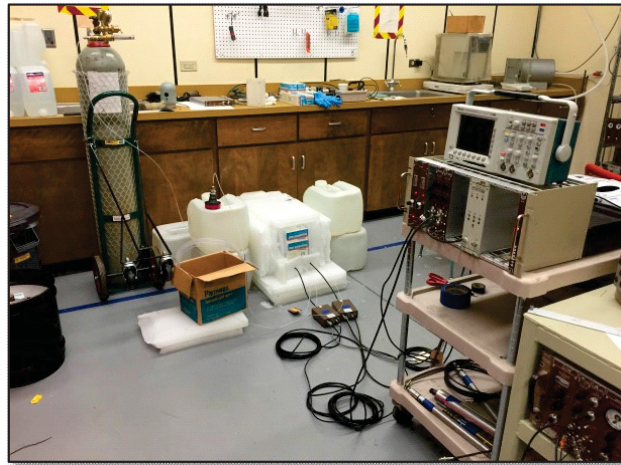


Figure 27. Fission material characterization at the Idaho State University Reactor Laboratory

3.3.3. HT MPFD Amplifier Electronics

The EDL at KSU had designed previous versions of amplifier electronics in support of MPFD deployments, however it was recognized that a more complex amplifier system was needed for future deployments so Mesytec was contacted to develop a 4 channel readout in support of future MPFD deployments. Initial V1.0 MPFD electronics have been tested with MPFDs at the KSU TRIGA to demonstrate proof of concept. Further refinements were added to develop an updated system as shown in Figure 28. The new V1.2 electronics design consist of a humidity resistant pre-amplifier box that interfaces directly with a NIM processing module. The V1.2 system includes charge sensitive preamplifiers, a HV bias supply, a filter stage and associated discriminators. All adjustments and data can be controlled and transmitted via USB interface. The electronics will output up to 4 discriminator signals for an external counter.

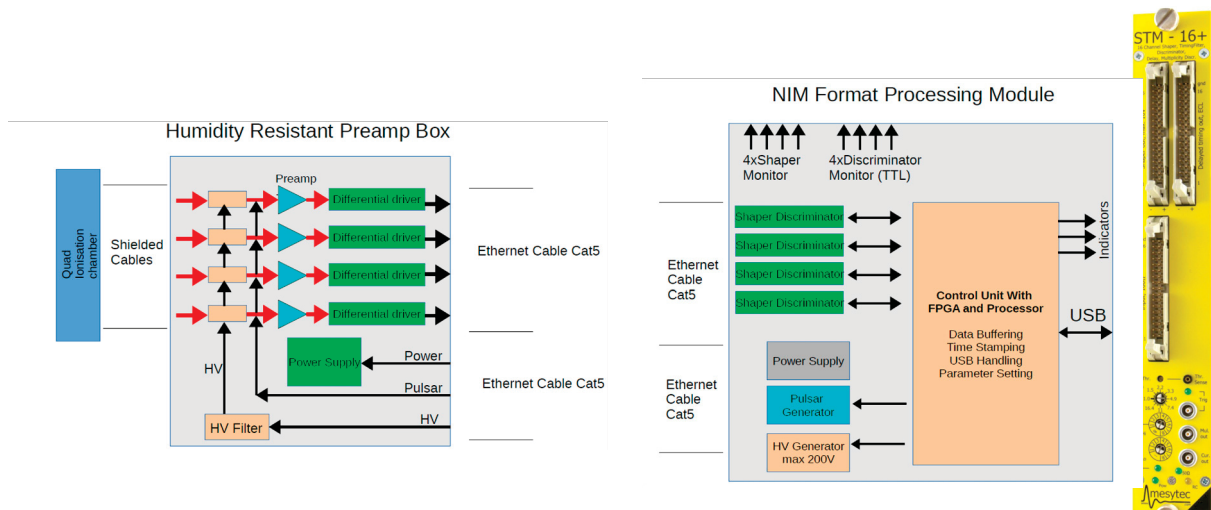


Figure 28. Mesytec electronics system schematics (left, middle) and processing module (right)

3.3.4. HT MPFD Fabrication

The HTTL's expertise in robust sensors for MTR irradiations was used to improve the design for the HT MPFD. Using this expertise, the round, stackable geometry that is more suitable for installation in a leak-tight tube and that has improved thermocouple connections and wiring transitions was maintained. Careful material selection should allow the HT MPFD to be capable of operating at temperatures up to 800 °C.

The main components of the HTMPFD detector and extension cable are specially manufactured hard-fired ultra smooth alumina substrates and crushable alumina insulators (Figure 29), respectively, all housed within leak-tight tubing. The outer sheath was changed from the stainless steel that was used in previous MPFD designs to Inconel for improved survivability in high temperatures and in harsh irradiation conditions. Wiring and tube-to-tube transitions use specially machined Inconel adapters that position the tubes uniformly for welding repeatedly. In addition, a tube cap was specially machined for the loose assembly tube for a tight fit. A "loose assembly construction" will be used with a rigid Inconel tube for the active area of the sensor (Figure 30). Modeling has shown that a wall thickness of 0.020" will allow the integrity of the sheath to survive in accident conditions up to 9000psi as required for transient irradiation testing. This tube contains hard-fired alumina substrates for the fission chamber electrodes and the fissile depositions and wire contacts. The crushable alumina insulators serve as an insulator for the six wire extension cable that also utilizes an Inconel outer sheath. In addition, the HT MPFD design moves the thermocouple to the extension cable to improve thermocouple response.

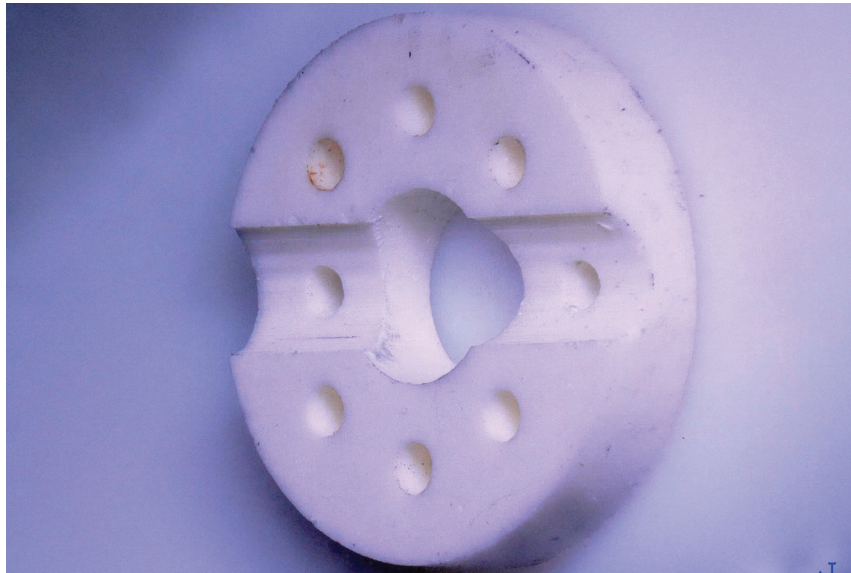


Figure 29. MPFD alumina substrates and insulators

HT MPFD construction used equipment and techniques perfected by INL's HTTL staff as for other in-pile sensors. The unique design of the MPFD requires several construction steps that aren't required for conventional in-pile sensor fabrication. Specifically, specialized techniques have been developed that emphasize robustness that include minimizing the number of wire splices and difficult to manufacture wiring transitions. The HT MPFD prototype was analyzed using the 2D X-ray scans to verify thermocouple placement in the extension wires prior to final assembly (Figure 31). After assembly was complete, a more detailed micro focus 3D computed tomography (CT) scan was completed at the HTTL to verify integrity of

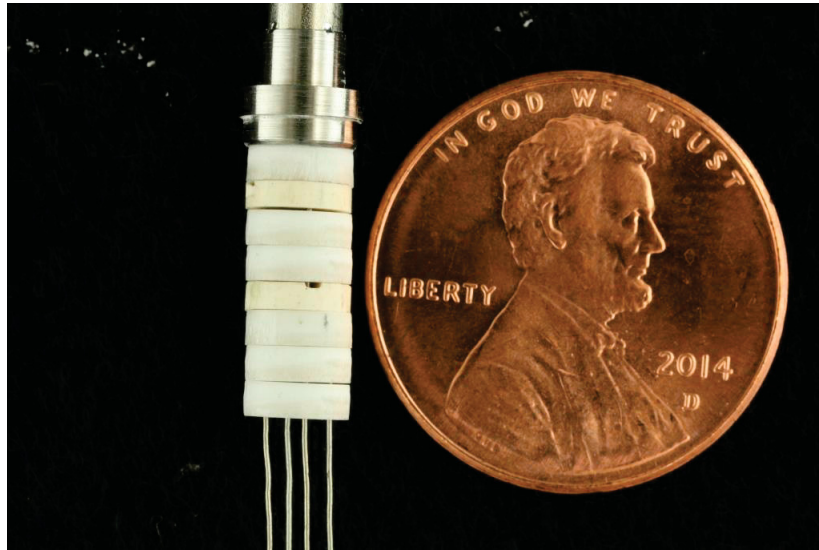


Figure 30. Extension cable with six wires threaded through insulation

the welds on the outer sheath (Figure 32). This scan was also used to identify accurate placement of the internal features of the HT MPFD prior to deployment.

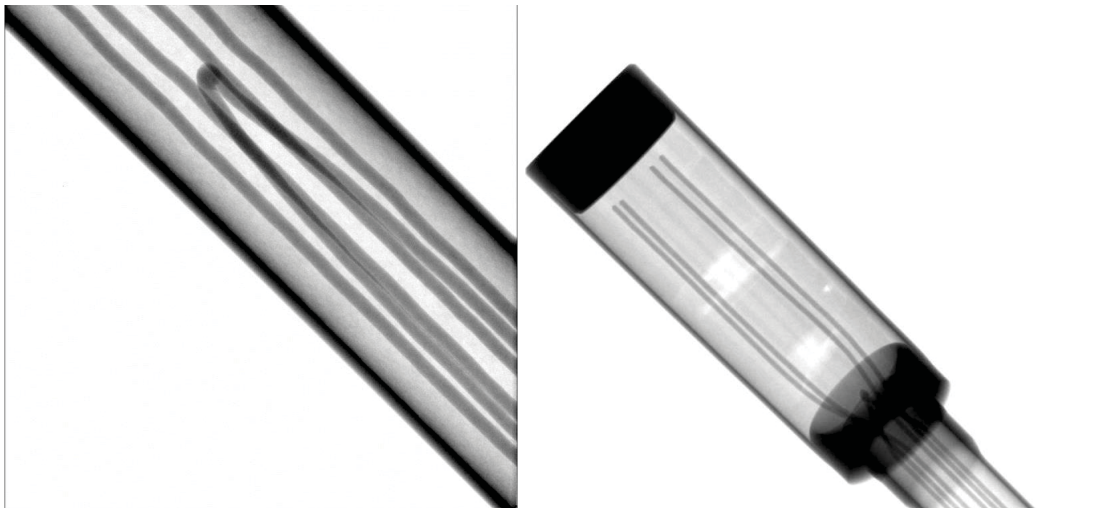


Figure 31. HT MPFD thermocouple placement in extension cable.

3.3.5. HT MPFD MCNP Modeling

Testing of experimental fuel in accident conditions is an important step in qualifying new reactor fuels and determining the safe operating conditions for advanced reactor designs. MPFDs will be deployed along with other advanced instrumentation in the Multi-SERTTA vehicle in the central test channel in the TREAT reactor as shown in Figure 33. The current MPFD design for tests at the TREAT facility was modeled in MCNP and tested in the neutron flux profile expected for that reactor. The MPFD sensitivities for ^{242}Pu , ^{232}Th , natural uranium, and 93% enriched ^{235}U were calculated and reported for the TREAT neutron flux profile. These results show that the expected interaction rate of the fissile coating of an MPFD

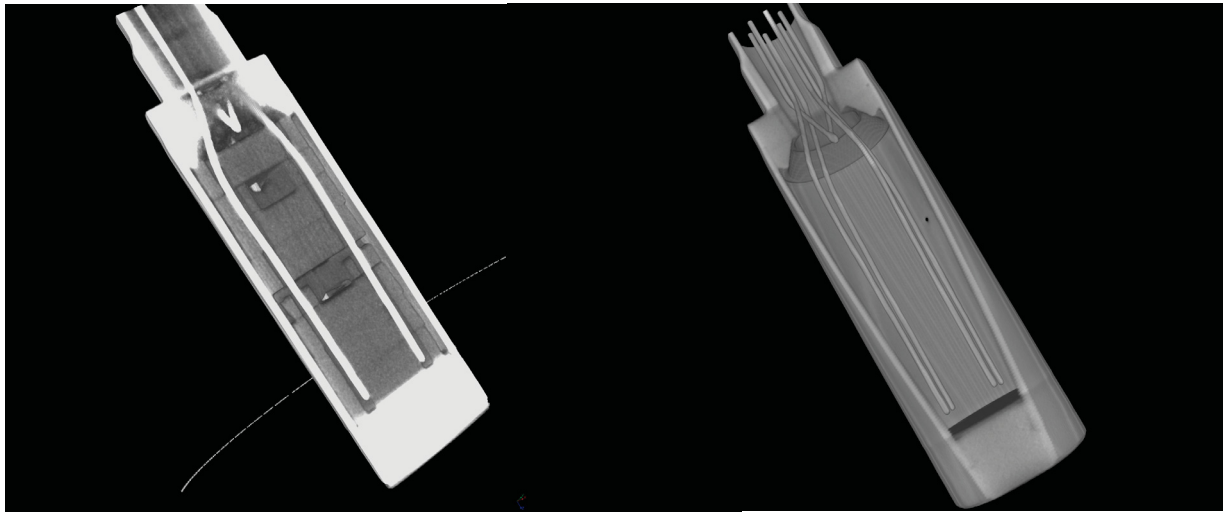


Figure 32. HT MPFD 3D CT scans with varying contrast ratios to identify internal features.

during peak flux levels of a TREAT pulse will exceed the operational range of traditional pulse-mode operation. Subsequently, current-mode electronics are being developed for transient testing of MPFDs. The variation of electrode thickness, active material thickness, and material were all examined. The parameters for the current MPFD design were found to have no negative effects on the operation of the sensor nor on the neutron flux in the vicinity of the experimental assembly.

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3.3.6. MPFD MTR Irradiation Opportunities

As previously stated, sensors developed for NEET ASI projects should strive to meet irradiation testing program needs. Two MTR irradiation programs have recognized the value of the NEET-developed MPFD technology and are independently funding irradiations of the MPFD technology. These are significant opportunities for the FY15-FY17 *NEET Enhanced Micro-Pocket Fission Detector for High Temperature Reactors* project because the irradiation expense of these experiments could not be included in the original project scope.

3.3.6.1. AGR 5/6/7 Irradiation

The NGNP program is pursuing evaluations of a suite of advanced real-time in-core sensors during their 3 year test campaign in the ATR. The AGR 5/6/7 irradiation program is funding the sensor procurement, data acquisition system and engineering effort required to integrate sensors into the test train (Figure 34). The suite of instrumentations includes:

- 1 Ultrasonic Thermometer (NEET-developed sensor)
- 1 Micro-Pocket Fission Detector (NEET-developed sensor)
- 3 Self Powered Neutron Detectors
- 2 High Temperature Ceramic Thermistors

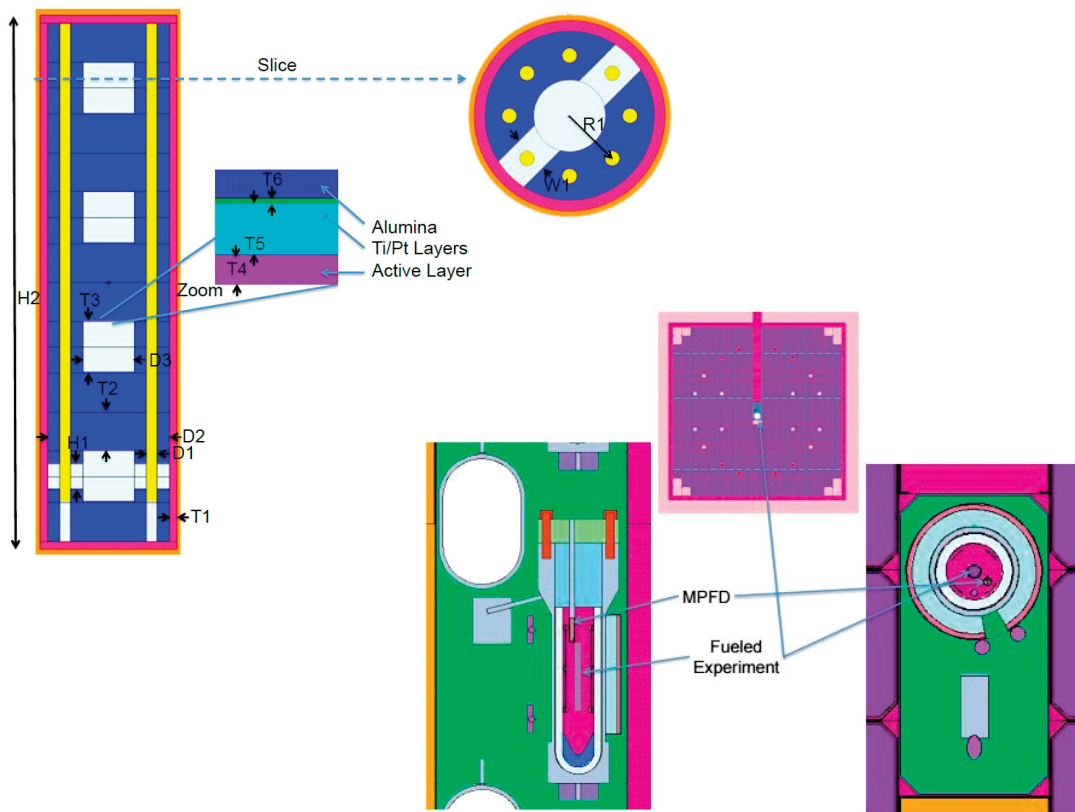


Figure 33. MCNP Transient-MPFD model (left) and location in experiment (right)

- 1 Fiber Optic Temperature Sensor
- 1 Ionization chamber

The program will use the HT MPFD design (Figure 22), but has requested a change from a Type K to a Type N thermocouple to match the other thermocouples installed in the test. Both the NEET-developed sensors and the other sensors in these evaluations will be tested at unprecedented flux levels by leveraging the AGR 5/6/7 irradiation program. This evaluation will further demonstrate the usefulness of the MPFD to simultaneously measure fast flux, thermal flux and temperature in a typical ATR long duration irradiation test.

3.3.6.2. Accident Tolerant Fuels

The Accident Tolerant Fuels (ATF) program is pursuing the MPFD technology for two irradiations. The first irradiation, ATF-2, will evaluate the a suite of advanced sensors, including the NEET-developed HT MPFD, in the ATR during a sensor qualification test. The ATF-2 irradiation program is funding the construction, installation and testing of a MPFD (along with other advanced sensors) during these irradiations. The irradiation isn't expected to be as long as the AGR 5/6/7 irradiation, but will still provide valuable insight into the operation of a MPFD in a typical ATR irradiation.

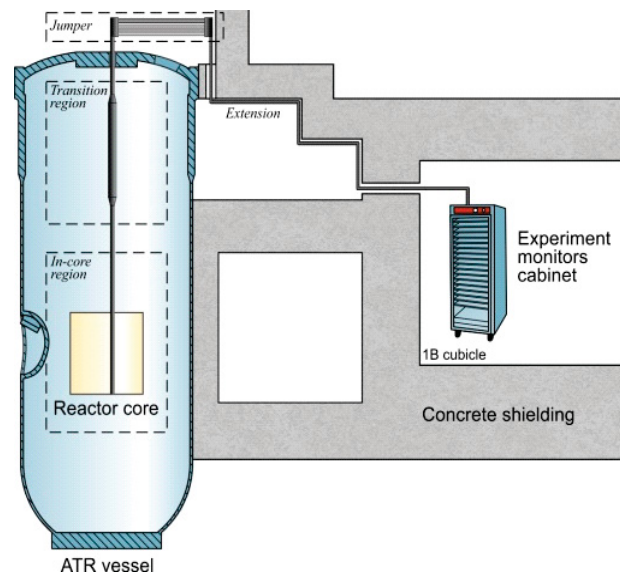


Figure 34. In-core sensor and equipment locations in the AGR 5/6/7 test

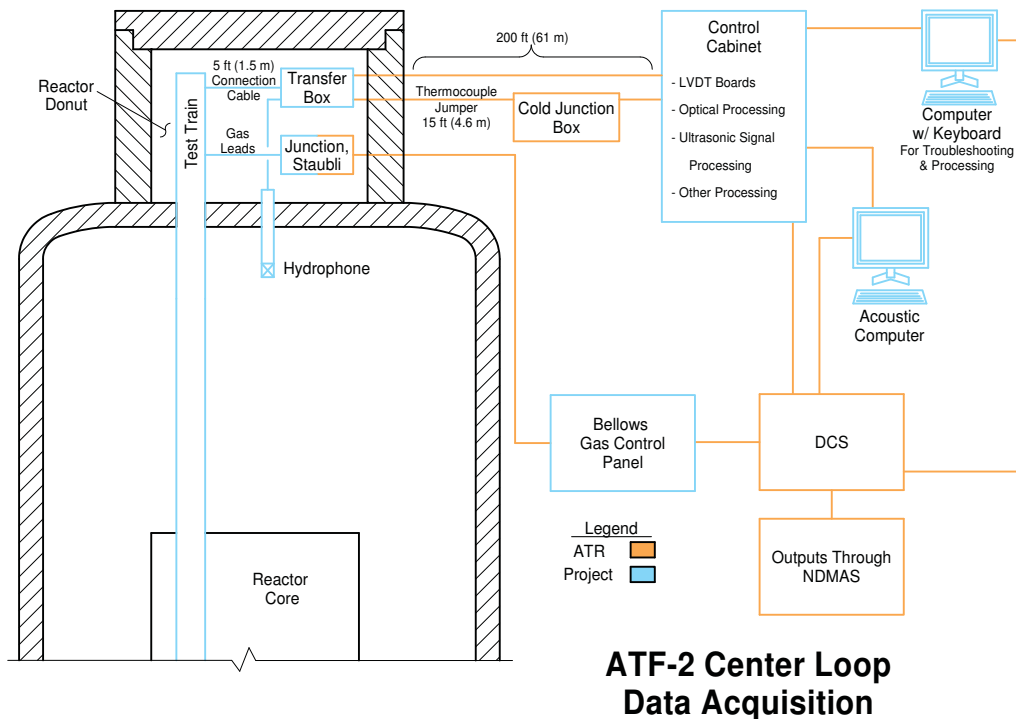


Figure 35. In-core sensor schematic for ATF-2 sensor qualification test

The second irradiation, ATF-3, will evaluate a specialized version of the MPFD technology that is suitable for TREAT transients. This version of the MPFD is scheduled to be installed in the multi Static Environment Rodlet Transient Test Apparatus (SERTTA) as shown in Figure 36 and Figure 37 to aid in the monitoring of experiment transients.³⁰ This effort will be a departure from previous MPFD designs because the focus will be on developing a robust, fast response MPFD that has two low efficiency and two high effi-

ciency neutron detectors and does not include a thermocouple as shown in Figure 38. Although the transient-specific MPFD design developed by ATF-3 will be different from other MPFD designs, it is expected that the information gained in the development of this transient-specific MPFD will provide insight into the developmental activities of the *NEET Enhanced Micro-Pocket Fission Detector for High Temperature Reactors* project. In addition, the information gained from initial ATF-3 irradiations will guide the future use of transient-specific MPFDs for TREAT irradiation testing for various DOE-NE programs requesting advanced instrumentation capabilities for transient testing.

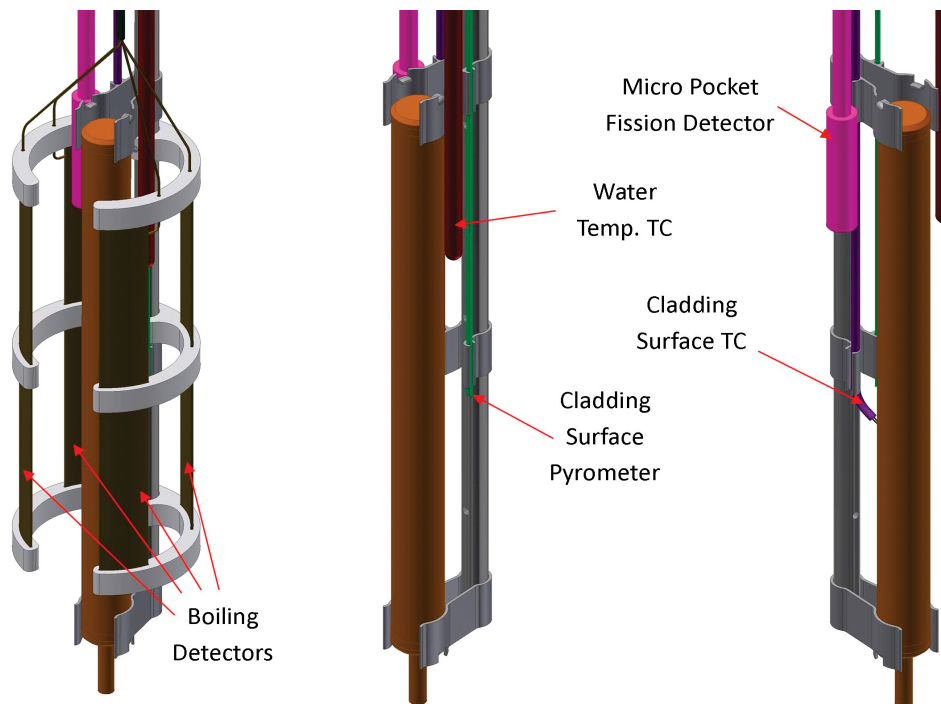


Figure 36. Sensor locations near the fuel specimen in the Multi-SERTTA irradiation test vehicle for ATF-3; MPFD shown in purple

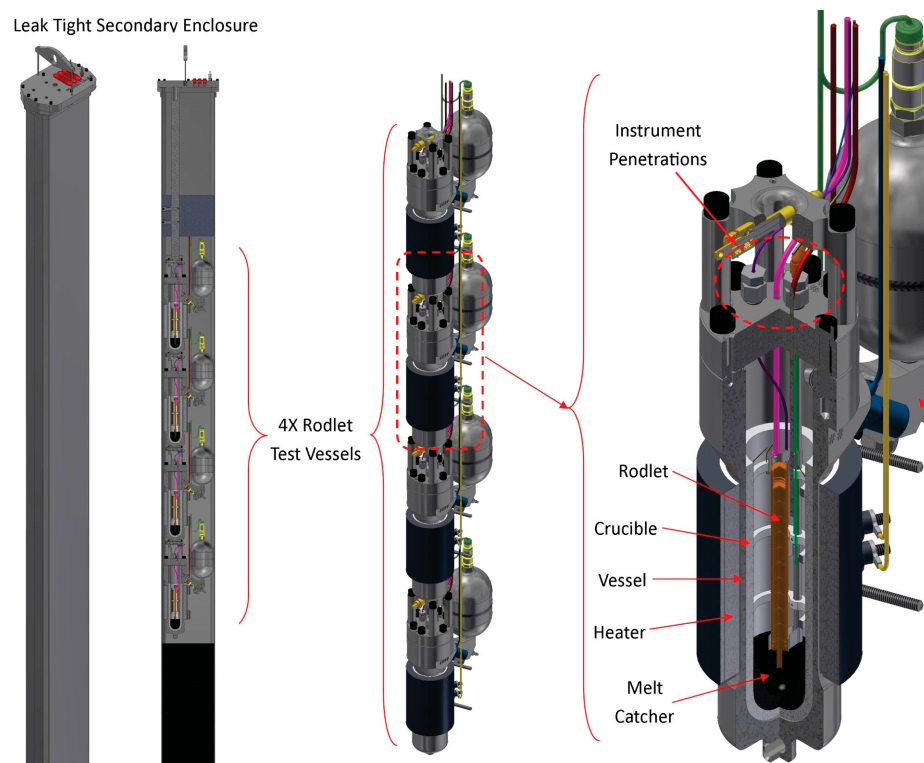
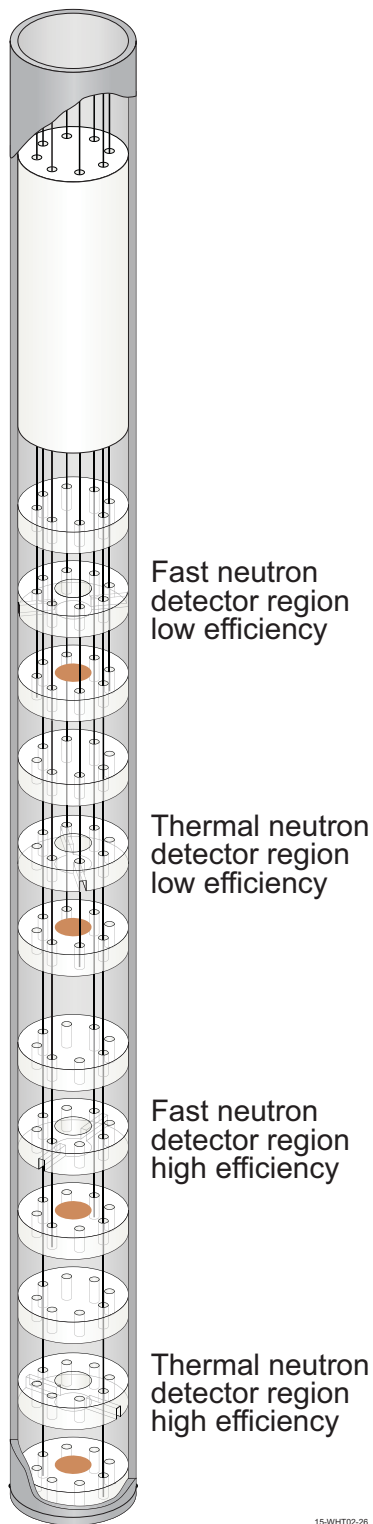


Figure 37. Overall layout of the Multi-SERTTA irradiation test vehicle for ATF-3; MPFD shown in purple on right



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Figure 38. Transient MPFD for ATF-3 multi-SERTTA TREAT irradiation

4. SUMMARY

All project objectives for year two of this three year project were met and all planned FY16 activities were completed. This section summarizes the objectives, accomplishments, importance, and future work of this NEET ASI project.

4.1. Project Objectives

The project objectives were carefully chosen to develop a sensor that will significantly advance flux detection capabilities for irradiation tests in US MTRs. The objectives for FY16 include:

- FY16 kickoff meeting at KSU.
- Refine MPFD design for high temperatures up to 800 °C.
- Select and procure new high temperature compatible materials for HT MPFD construction.
- Refine MPFD construction methods for high temperatures up to 800 °C.
- Refine amplifier design to be compatible with the HT MPFD.
- Refine electrode deposition for high temperatures up to 800 °C.
- Refine fissile material deposition system and methods for HT MPFD fissile deposition.
- Interface with various DOE-NE programs interested in HT MPFD irradiation testing.
- Submit FY15 project report and publications on MPFD development

4.2. Accomplishments

The research completed in year one of this three year project is well on its way to providing an robust sensor for DOE-NE irradiation testing programs. Highlights from FY16 research accomplishments include:

- A joint collaboration was initiated between INL, KSU, and CEA. Note that CEA is participating at their own expense because of interest in this unique new sensor.
- A project meeting was held at KSU to discuss the roles and responsibilities between INL and KSU for development of the HT MPFDs.
- An updated HT MPFD design was developed.
- Selected and procured new high temperature compatible materials for HT MPFD construction.
- Construction methods to support the new design were evaluated.
- Performed laboratory evaluations of construction methods of HT MPFD.
- Electrical contact and fissile material plating performed.
- Updated detector electronics evaluations.
- Publication of MPFD technology accepted.
- Provide input to NGNP AGR 5/6/7 irradiation program for installation of HT MPFD in irradiation test.
- Provide input to ATF-2 irradiation program for installation of HT MPFD in sensor qualification test.
- Provide input the ATF-3 irradiation program for installation of a transient MPFD design in the multi-SERTTA irradiation test vehicle.

As noted within this report, this is year one of a three year project. Initial work is underway with future work scheduled to finish development and evaluate the HT MPFD in a variety of testing conditions. In addition to the evaluations planned in this project, the HT MPFD will be installed in ATR and TREAT irradiation tests as part of other DOE-NE programs evaluating the performance of candidate new fuels and materials to better characterize irradiation test conditions.

4.3. Importance of Superior Flux Detector

When implemented, these sensors will significantly advance flux detection capabilities for irradiation tests in US MTRs. Specifically, deployment of HT MPFDs will address several challenges faced in irradiations performed at MTRs:

- Current fission chamber technologies do not offer the ability to measure fast flux, thermal flux, and temperature within a single compact probe; HT MPFDs offer this option.
- HT MPFD construction is very different than current fission chamber construction; the use of high temperature materials and unique design characteristics allow HT MPFDs to be specifically tailored to survive harsh conditions encountered in high performance MTR irradiation tests.
- The higher accuracy, high fidelity data available from the compact HT MPFD will significantly enhance efforts to validate new high-fidelity reactor physics codes and new multi-scale, multi-physics codes.
- HT MPFDs can be built with variable sensitivities to survive the lifetime of an experiment or fuel assembly in some MTRs, allowing more efficient and cost effective power monitoring over a wide range of reactor powers.
- The small size of the MPFD technology allows multiple sensors to be deployed, offering the potential to accurately measure flux and temperature profiles in the reactor.

4.4. Future Work

Further research to be completed in FY17 will focus on finalizing construction and evaluating the HT MPFD in a variety of testing conditions. These conditions include continued high temperature furnace and autoclaves evaluations up to temperatures of 800 °C. Material and construction integrity will be verified along with accompany electronic readout reliability. In addition, analysis models for sensor performance will be developed and the HT MPFD evaluations will be used to benchmark the analysis models. The HT MPFD design will be further refined as needed and prior evaluations will be repeated to compare and contrast refinements.

In addition to the development of the HT MPFD in this NEET ASI project, other DOE-NE irradiation programs are requesting MPFDs to be included in their irradiation tests. The NGNP program is actively pursuing instrumenting the graphite in the top capsule in an upcoming fuel irradiation, AGR-5/6/7, with a HT MPFD. This evaluation will further demonstrate the usefulness of the MPFD technology to simultaneously measure fast flux, thermal flux and temperature in an typical ATR long duration irradiation test. This is a significant opportunity for the HT MPFD project because the irradiation expense of a long-term MTR experiment could not be included in the original project scope.

The Accident Tolerant Fuels (ATF) program is pursuing the MPFD technology for two irradiations in the ATF-2 and ATF-3 irradiation programs. The first irradiation will evaluate a HT MPFD in the ATR during a sensor qualification test. The ATF-2 irradiation program is funding the construction, installation and testing of a these HT MPFDs (along with other advanced sensors) during these irradiations. The second irradiation will evaluate a specialized version of the MPFD technology that is suitable for TREAT transients. This version of the MPFD is scheduled to be installed in the multi SRERTTA irradiation test vehicle for calibration of irradiation conditions. Although the transient-specific MPFD developed by ATF-3 will be different from the HT MPFD design, it is expected that the information gained in the irradiation of a transient-specific MPFD will aid in developing a robust HT MPFD that can survive a variety of irradiation conditions.

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