

Publicly Available Molten Salt Data/Benchmarks

April 2024

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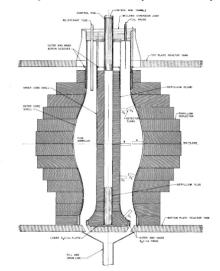
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Publicly Available Molten Salt Data/Benchmarks

Three Molten Salt Experiments Were Operated at ORNL

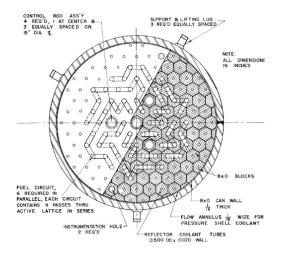
Zero Power Reactor Experiment (ZPRE)

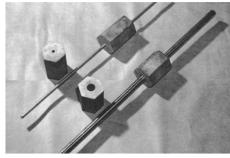




NaF-ZrF4-UF4 fuel, Be moderator, Inconel vessel.

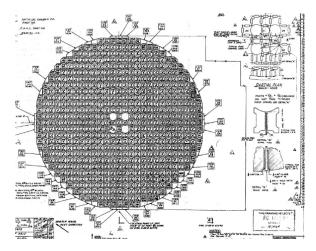
Aircraft Reactor Experiment (ARE)





NaF-ZrF4-UF4 fuel, BeO moderator (+Na), Inconel vessel.

Molten Salt Reactor Experiment (MSRE)

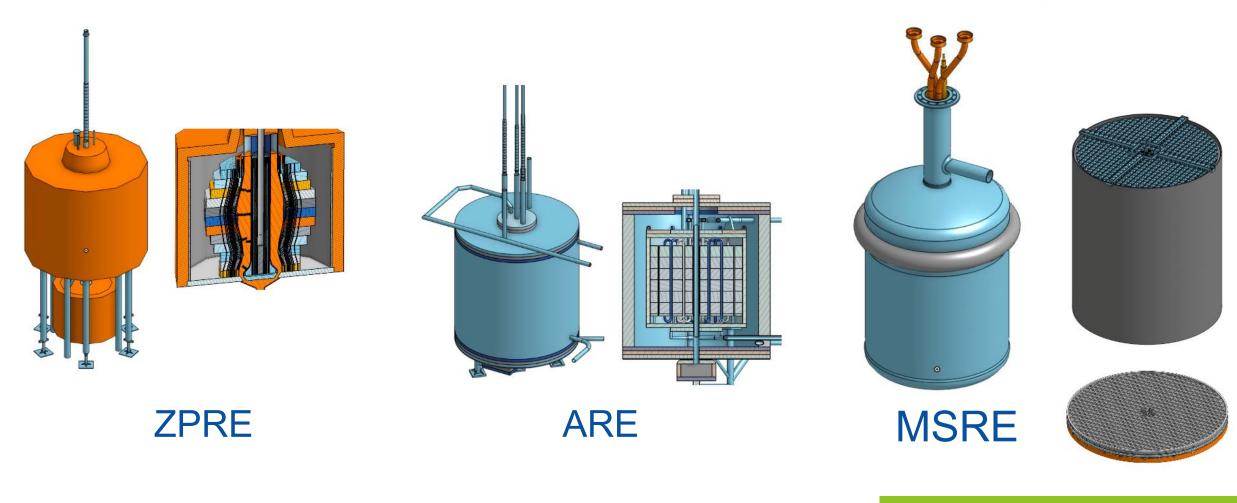




7LF-BeF2-ZrF4-UF4 fuel, C moderator, hastelloy vessel.

Copenhagen Atomics Has Created Open-Source Models





Copenhagen Atomics Open-Source MSR Modeling

from CAD to burnup or dose calculations



Open-source Monte Carlo neutron and photon transport





CAD meshing tool

☐ openmsr / CAD_to_OpenMC



 Installation scripts (OpenMC + GAGMC + MOAM + Embree + CAD_to_OpenMC + nuclear data)

Package Details: openmc-git v0.13.2.r497.g6218becb1-1

Git Clone URL:

https://aur.archlinux.org/openmc-git.git (read-only, click to copy)

Package Base:

openmc-git

MSR modeling capabilities being build into OpenMC

msr continuous capabilities #2358



church89 wants to merge 132 commits into openic-devideve top from openismins 18.2 cont (

Copenhagen Atomics Assessment of Data Quality

• ZPRE:



- Core design accuracy is decent
- Zero power data only, but only reactor with a Be metal reflector and unique design
- Experimental data are sparse

ARE:

- Core design accuracy is decent
- External design accuracy is poor
- Moderate data
- Experimental data is sparse

MSRE

- Core design accuracy is good
- External design accuracy is decent
- Experimental data is decent but sparse
- In general, access to many drawings, data logs, and internal reports, not just high level ORNL & ORNL-TM reports

Copenhagen Atomics Assessment of Benchmark Potential



- ZPRE: criticality benchmark decent
- ARE:
 - Criticality benchmark decent (includes some β_{eff} data)
 - Transient benchmark possible (only one case)
 - Missing data for burnup/redox benchmark
- MSRE
 - Criticality benchmark decent (includes β_{eff} data)
 - Transient benchmark decent (limited cases)
 - Burnup/redox benchmark decent (missing power history for ²³³U & Pu run)
- General comment: nuclear data files are optimized for solid fuel, so deviations are expected.

Other Copenhagen Atomics Activities



Ongoing

 Actively reaching out and helping other teams (university, national lab, and MSR developers) to learn OpenMC and develop open-source capabilities.

Upcoming

- Planning to run a subcritical test reactor in 2025, we plan to turn this into a open-source benchmark.
- Planning to run a 1MW_{th} test reactor for 30 days by 2025/2026, and plan to turn this into an open-source benchmark.
- Will dedicate more resources to developing the ORNL benchmarks in 2024.
- Repositories updated as needed

Github Repositories

ZPRE: https://github.com/openmsr/zpre

ARE: https://github.com/openmsr/are

MSRE: https://github.com/openmsr/msre



 These links, tools mentioned and archived data are available at https://github.com/openmsr



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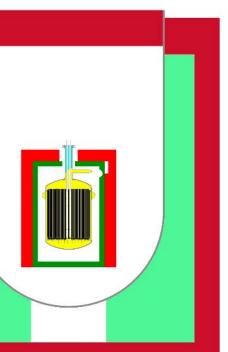
MSRE Model



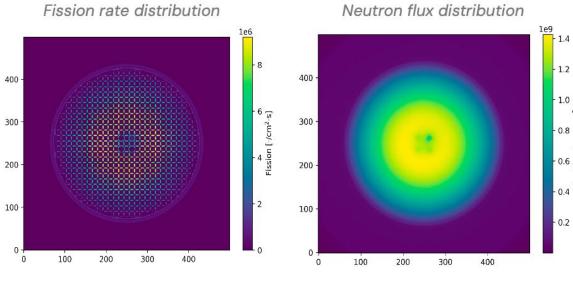
CAD geometry



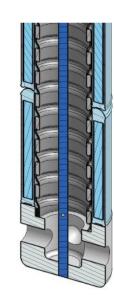
Mesh geometry

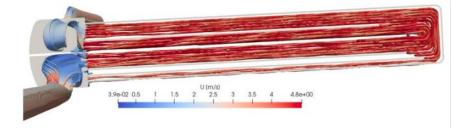


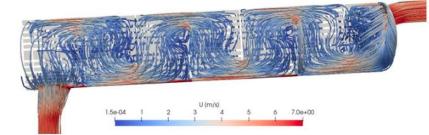
Fission rate distribution



Neutrons flux spectrum





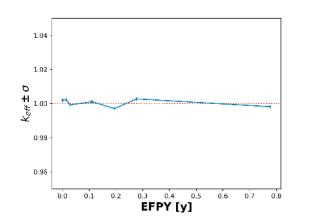


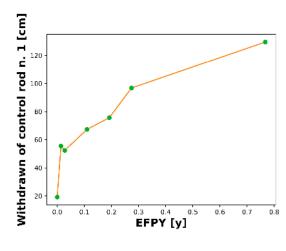


primary heat exchanger CFD

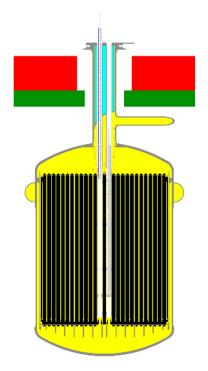
MSRE Burnup

- Depletion power: 8 MWth
- Total fuel salt mass: 4590 kg
- U235 initial load: 65.25 kg
- Removal rates:
- Noble gas (Xe, Kr) 4.067e-5 1/s
- Metals (Se, Nb, Mo, Tc, Ru, Rh, Pd, Ag, Sb, Te) 8.777e-3 1/s

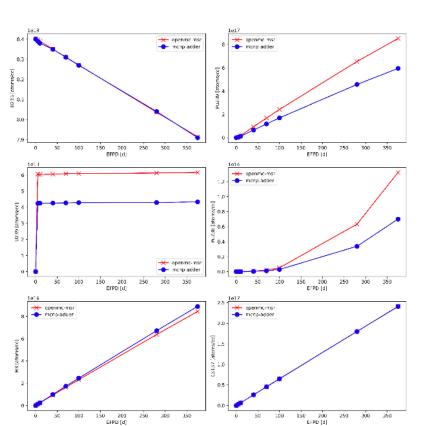


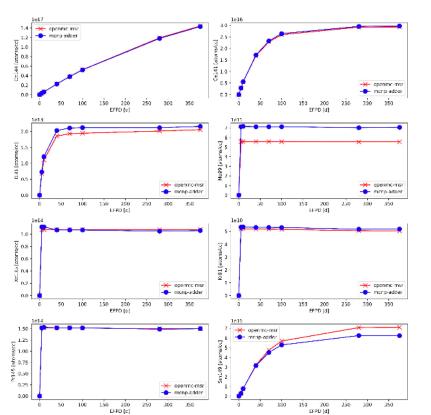


Code-to-code benchmark









MSRE Burnup

