



Presentation: Reactor Containment Passive Safety Analysis: Steam Condensation in Presence of Non-condensable Gas Scaled Experiment and Modeling

October 2024

Changing the World's Energy Future

Palash Kumar Bhowmik, Joshua P. Schlegel, Piyush Sabharwall



INL is a U.S. Department of Energy National Laboratory operated by Battelle Energy Alliance, LLC

DISCLAIMER

This information was prepared as an account of work sponsored by an agency of the U.S. Government. Neither the U.S. Government nor any agency thereof, nor any of their employees, makes any warranty, expressed or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness, of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. References herein to any specific commercial product, process, or service by trade name, trade mark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the U.S. Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the U.S. Government or any agency thereof.

Presentation: Reactor Containment Passive Safety Analysis: Steam Condensation in Presence of Non- condensable Gas Scaled Experiment and Modeling

Palash Kumar Bhowmik, Joshua P. Schlegel, Piyush Sabharwall

October 2024

**Idaho National Laboratory
Idaho Falls, Idaho 83415**

<http://www.inl.gov>

**Prepared for the
U.S. Department of Energy
Under DOE Idaho Operations Office
Contract DE-AC07-05ID14517**



**Pacific
Basin
Nuclear
Conference**

PBNC 2024

Reactor Containment Passive Safety Analysis: Steam Condensation in Presence of Non-condensable Gas Scaled Experiment and Modeling

Palash K. Bhowmik, *R&D Staff, Reactor System
Design and Analysis, INL*

Co-authors: Joshua P. Schlegel (Missouri S&T)
and Piyush Sabharwall (INL)

Introduction: Problem Overview

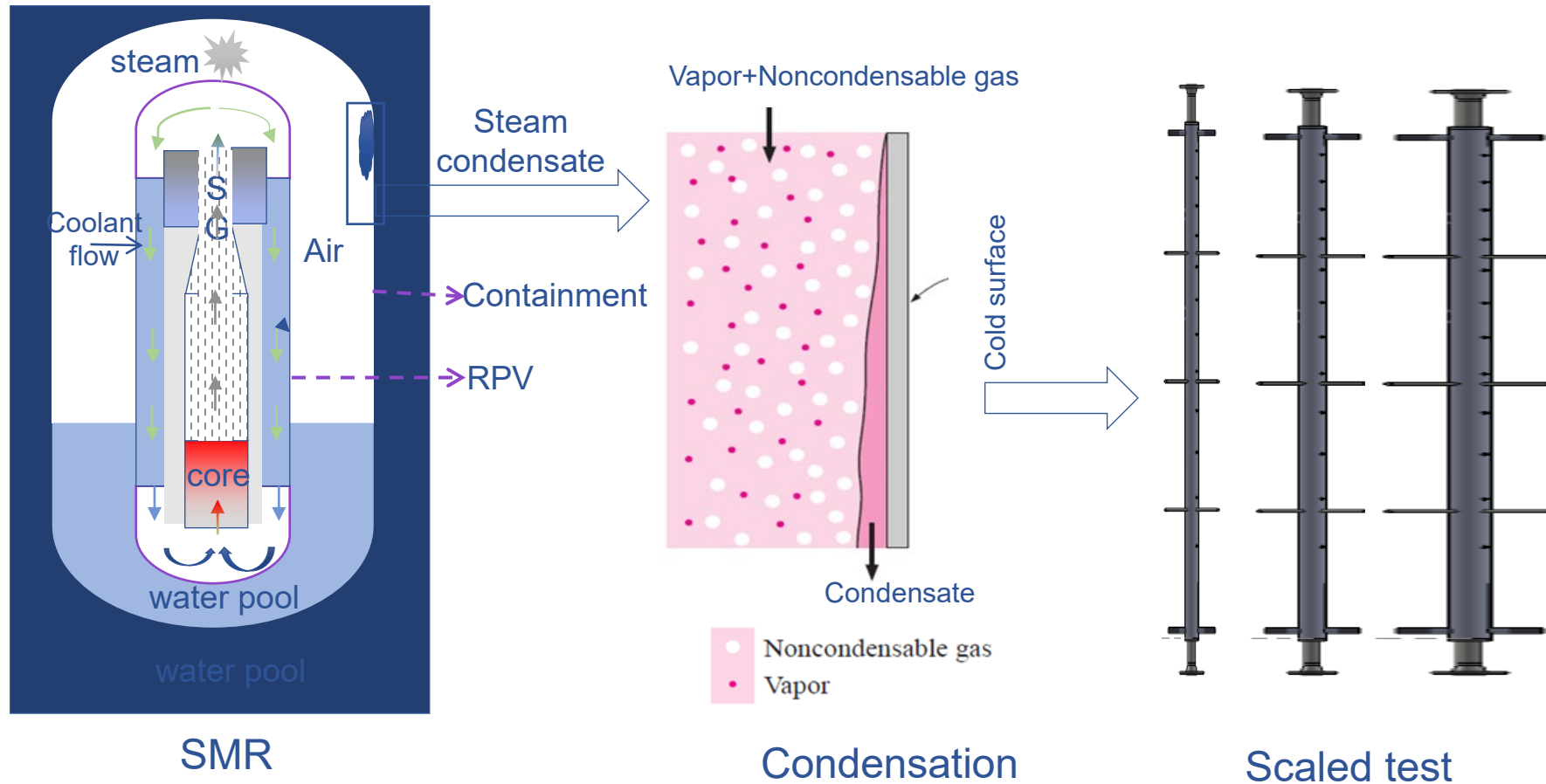


Fig. 1: Overview of the condensation scaled separate effect experiment.

Introduction: Objectives and Motivations

- Small modular reactors (SMRs) have attractive features, such as factory-built construction, modular design, easier transportation, and multi-units capacity addition.
 - Worldwide, over 80 different SMRs are currently in the design and development stage, with most being light-water-cooled [1-2].
- However, the verification and validation of the required models is a must for obtaining regulatory approval, which supports
 - evaluation model development and the assessment process for the reactor system, similar to the passive containment cooling system (PCCS) for various loss-of-coolant accidents (LOCAs) [3–4].
- The previous condensation heat transfer (CHT) works were grouped into theoretical, experimental, and numerical studies [5].
- The experimental studies were subgrouped into separate and integral effect tests with a wide range of varying geometric, physics, fluids, and operating conditions [6].
- Similarly, the theoretical and numerical studies were subgrouped into conceptual modeling, simulations, and multiphysics-computational fluid dynamics (CFD) using commercial software, system, and in-house-developed codes [7].
- Many of the earlier studies on the reactor in containment condensation considered the effect of non-condensable gases (NCGs), such as air, nitrogen, hydrogen, and helium [8–13].
- PCCSs are especially important for SMRs, due to their compact nature. SMRs usually incorporate suppression-type, submerged, or air-cooled PCCSs.

Introduction: Objectives and Motivations (cont'd)

- Previous PCCS CHT experimental studies mostly focused on
 - small and fixed geometric (mostly using a 2-in. pipe) and testing conditions causes challenges to geometric scaling and operational condition relevant to the prototypic conditions for SMRs [5–6].
- Therefore, a scale test dataset and simplified modeling approach is required in most cases for early-stage reactor system design and analysis.

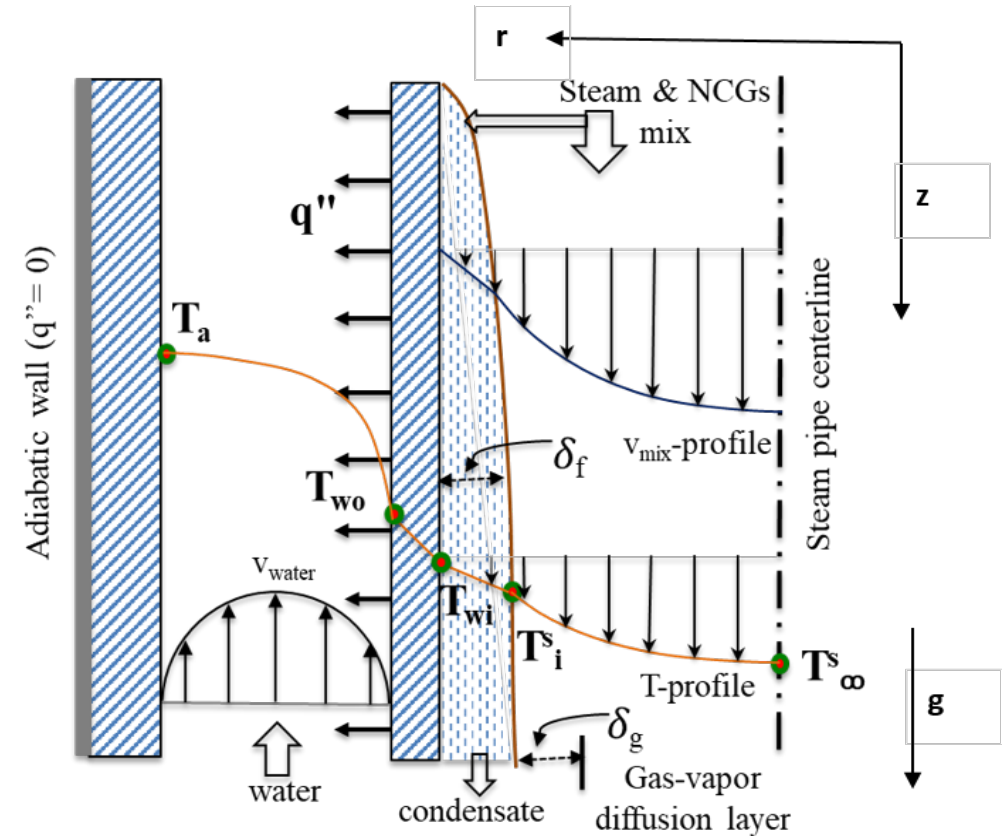


Fig. 2: Schematics of CHT: physical phenomena [14].

Experimental Facility

- This study utilized three scaled test sections
 - 1-, 2-, and 4-in.-diameter steam condensers with annular/jacket cooling of 2-, 3-, and 6-in.-diameter tubes
 - to obtain steam condensation test data (mostly axial temperature) in the presence of NCG (i.e., nitrogen),
 - varying steam-NCG mix flow rates, and annular cooling flow rates.

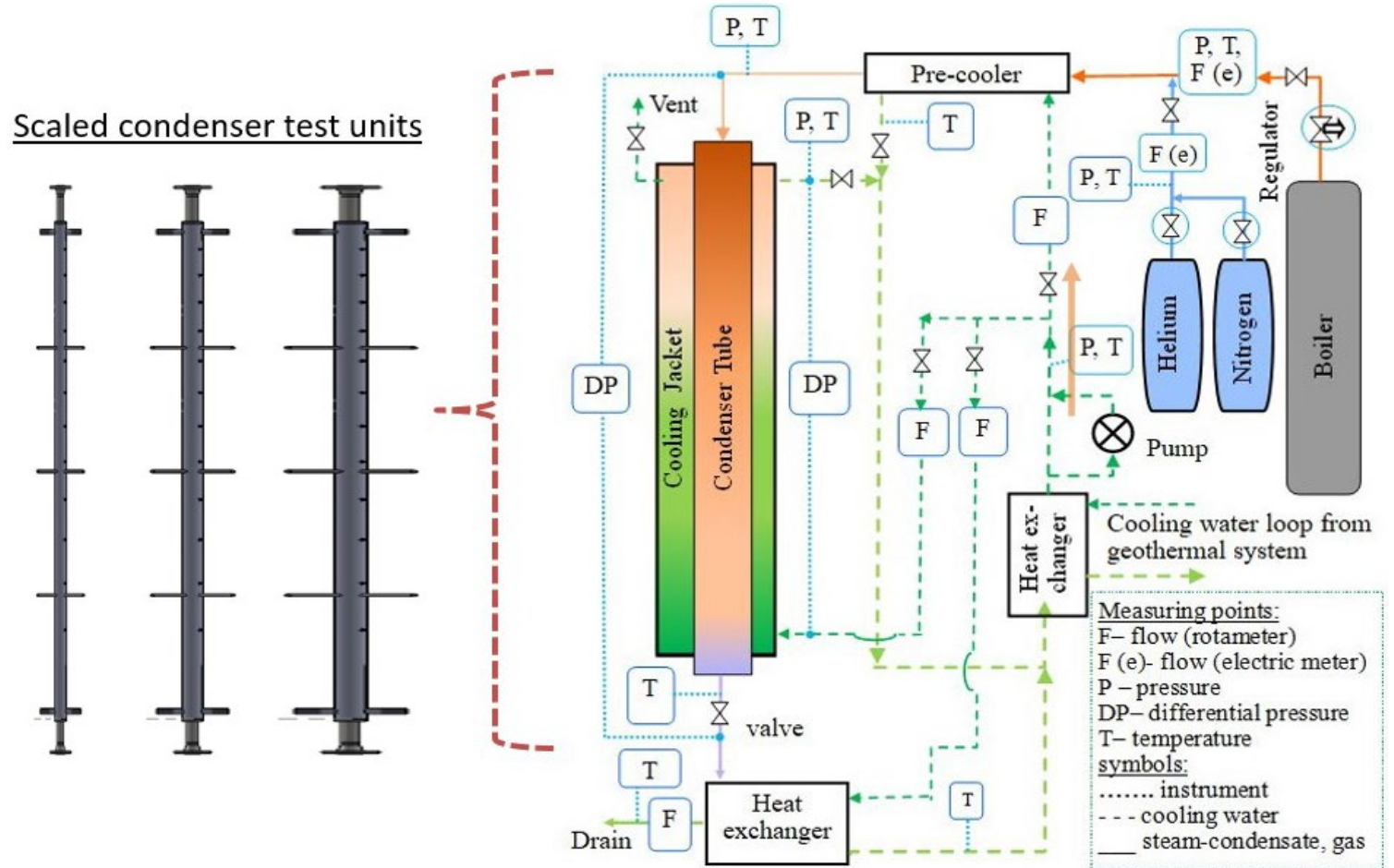


Fig. 3: Final test facility instrument and control schematic [12].

Data Reduction Method

- The test data reduction method for CHT, consists of the following three primary stages of estimating parameters:
 - Estimating coolant bulk temperature (T_b) and local heat flux (q'')
 - Local HTC, blowing parameter, and film thickness
 - Dimensionless parameters: Reynolds (Re) and Nusselt (Nu) numbers
 - Uncertainty quantification

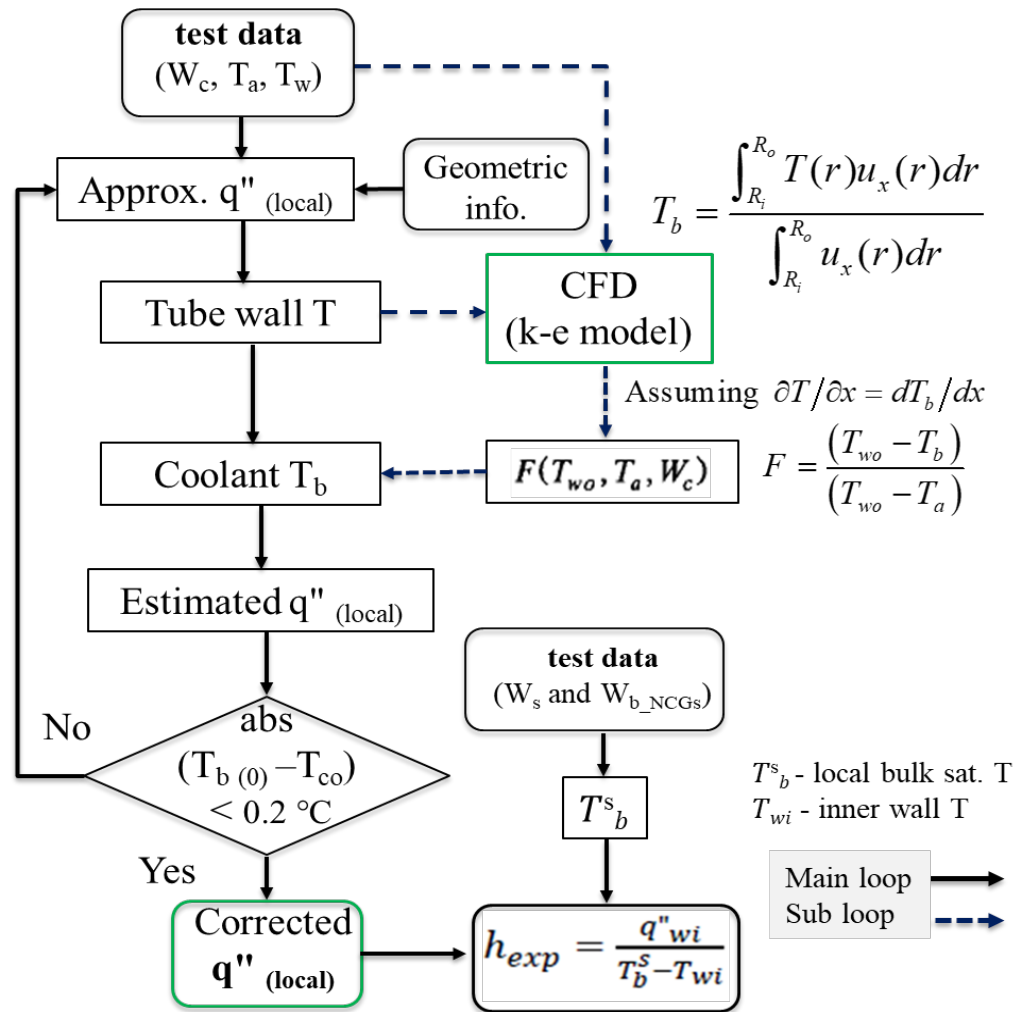


Fig. 4: Local heat flux and HTC estimation [14].

Models and Equations

Parameters	Models and Equations
T_b and local q''	$q''_{wi}(z)_{\text{approx.}} = -\frac{W_c c_p}{\pi d_i} \frac{dT_a(z)}{dz} \text{ and } T_{wi} = T_{wo} + \frac{r_i q''_{wi}(z) \ln\left(\frac{d_o}{d_i}\right)}{k_w}; \quad q''_{wi}(z) = -\frac{W_c c_p}{\pi d_i} \frac{dT_{b,c}(z)}{dz}$ <p>where, $T_{b,c}$, T_{wi}, T_{wo}, and T_a, are the temperature of the coolant (bulk), condensing tube inner wall, condensing tube outer wall, and jacket water cooling. W is the mass flow rate, z is the axial length, d for diameter, k for thermal conductivity, c_p for specific heat capacity; subscript c for coolant.</p>
Local HTC, blowing parameter β, and film thickness δ	$h_{\text{exp}} = \frac{q''_{wi}}{T_b - T_{wi}}, \text{ and } \Gamma = \frac{g}{\mu} \rho_1 (\rho_1 - \rho_m) \frac{\delta_f^3}{3} + \frac{\rho_1 \tau_i \delta_f^2}{2 \mu_1}; \quad \tau_i = 0.5 f_{io} \rho_m (u_m - u_i)^2 \frac{\beta_f}{\exp(\beta_f) - 1}; \quad \beta_f = \frac{m''}{\rho_m u_m f_{io}/2} \text{ and } \delta_{fo} = \left(\frac{3 u_1 \Gamma}{g \rho_1 (\rho_1 - \rho_m)} \right)^{1/3}$ <p>where, h for HTC, δ for film thickness, Γ for liquid flow per unit perimeter, g for gravity, μ for dynamic viscosity [kg/m s], τ interfacial shear stress; and subscript i and l for inner/interface and liquid, respectively.</p>
Dimensionless numbers: Re and Nu	$\frac{Re_f}{1 - \left(\frac{\rho_m}{\rho_1}\right)} = \frac{\delta_f^{*3}}{3} + \frac{\tau_i^* \delta_f^{*2}}{2}; \quad Nu_f = \frac{h_f L}{k_c} = (Nu_{f,h}^4 + Nu_{f,tu}^4)^{1/4}; \quad Nu_{f,la} = \frac{1}{\delta_f^*} \text{ and } Nu_{f,tu} = a Re_f^b Pr^c (1 + e \tau_i^{*f})$ <p>where, Re for Reynolds number, Nu for Nusselt number, Pr for Prandtl number, and L for characteristics length. Subscript f for film, m for mix, i for interface, tu for turbulent.</p>
Dimensionless Numbers: Re and Nu	$\frac{Re_f}{1 - \left(\frac{\rho_m}{\rho_1}\right)} = \frac{\delta_f^{*3}}{3} + \frac{\tau_i^* \delta_f^{*2}}{2}; \quad Nu_f = \frac{h_f L}{k_c} = (Nu_{f,h}^4 + Nu_{f,tu}^4)^{1/4}; \quad Nu_{f,la} = \frac{1}{\delta_f^*} \text{ and } Nu_{f,tu} = a Re_f^b Pr^c (1 + e \tau_i^{*f})$ <p>where, Re for Reynolds number, Nu for Nusselt number, Pr for Prandtl number, and L for characteristics length. Subscript f for film, m for mix, i for interface, tu for turbulent.</p>
Uncertainty quantification	$\frac{\sigma_{\text{exp}}}{h_{\text{exp}}} = \left[\left(\frac{\sigma_{w_{cw}}}{W_{cw}} \right)^2 + \left(\frac{\sigma_{c_p}}{c_p} \right)^2 + \left(\frac{\sigma_{d_i}}{d_i} \right)^2 + \left(\frac{\sigma_{(T_c - T_w)}}{(T_{\text{sat}} - T_{wi})} \right)^2 + \left(\frac{\sigma_{(cw/dx)}}{d T_{cw/dx}} \right)^2 \right]^{1/2} \text{ and } \frac{\sigma_f}{f} = \left[\left(\frac{\sigma_{h_{\text{exp}}}}{h_{\text{exp}}} \right)^2 + \left(\frac{\sigma_{h_{Nu}}}{h_{Nu}} \right)^2 \right]^{1/2}$ <p>where, σ for error, dx for node length, sat for saturation, exp for experiment.</p>

Table I: Data reduction method and the relevant models/equations [14].

Models and Equations (cont'd)

Parameters	Semi-empirical Models/Correlations
Vierow (1990) [15]	$f = \frac{h_{exp}}{h_{Nu}} = f_1 \cdot f_2 = \left(1 + aRe_{mix}^d\right) \cdot \left(1 - bM_{NCGs}^c\right)$ <p>where, f with f_1, including the effect of interfacial shear (δ_{shear}) and surface waviness to improve the film heat transfer.</p>
Kuhn et al. (1996)[16]	$f = \frac{h_{exp}}{h_{Nu}} = \frac{\delta_{shear}}{\delta_{Nu}} \cdot \left(1 + a\left(\frac{Re_f}{4}\right)^d\right) \cdot (1 - bM_{NCGs}^c)$ <p>This is the modified Vierow (1990) f with f_1, including the effect of interfacial shear (δ_{shear}) and surface waviness. The details parameter information of f is available in Table 2 of Bhowmik et al. (2022) [6].</p>
Park and No (1999) [17]	$f = \frac{h_{tot}}{h_f} = 0.0012 W_{nc}^{-1.4} Ja^{-0.63} Re_f^{0.24}$ <p>for $1715 < Re_g < 21670, 0.83 < Pr_g < 1.04, 0.111 < M_{air} < 0.836$, $0.01654 < Ja < 0.07351$, and $12.4 < Re_f < 633.6$.</p> <p>where, dependence of steam-NCGs mixture Re and Prandtl number, Pr on CHT and developed f using gas mass fraction, Jacob number, Ja and liquid film Re.</p>
Lee and Kim (2008) [18]	$f = \tau_g^{*0.3124} (1 - 0.964M_a^{0.402})$ <p>for $0.06 < \tau_g^* < 46.65, 0.038 < M_{air} < 0.814$</p> <p>where, f for steam-NCGs mixture in a U-tube in a reflux condensation, using gas mass fraction and shear force of the mixture.</p>

Table II: Semi-empirical models/equations [6]

Test Data

- Scaled experiments were conducted by applying
 - Steam and steam-NCG mixtures to three different test sections.
 - Nitrogen served as the NCG. Test data were collected for varying
 - steam-NCG mixture mass fractions (M, %),
 - steam-NCG mass flow rates, and
 - coolant flow rates.
 - Figure 5, Figure 6, and Figure 7, respectively, present representative sample
 - test datasets for the 4-in. test section, collected under a wide range of NCG mass flow rates (i.e., high, moderate, and low).
 - Tests A-run0.9N0a, A-run2.1N4, and A-run2.1N8 represent the high (i.e., 13.22 kg/hr), moderate (i.e., 8.08 kg/h), and low (i.e., 4.41 kg/h) NCG:N₂ flow cases, respectively.

Test Data (cont'd)

Test Section	Steam flow	Pin	Tin	NCF: N ₂ flow	M
1-in	30-53	109.6-109.9	100.2-103	2.9-14.7	5.5-23.6%
2-in	30.7-42.2	111.4-115.7	102.6-105.4	2.6-12.9	6.1-31.6%
4-in	54.7-60.1	140.4-150.4	99.4-103.5	4.4-3.2	7-18.1%

Table III: Selected steam-N₂ mix condensation test conditions/ranges.

Note: Here, TS for test section, inlet mass flow rate (F [kg/hr]) for steam and NCG[N], inlet pressure (Pin [kPa]), inlet temperature (Tin [° C]), and NCG mass fraction (M, %).

Results

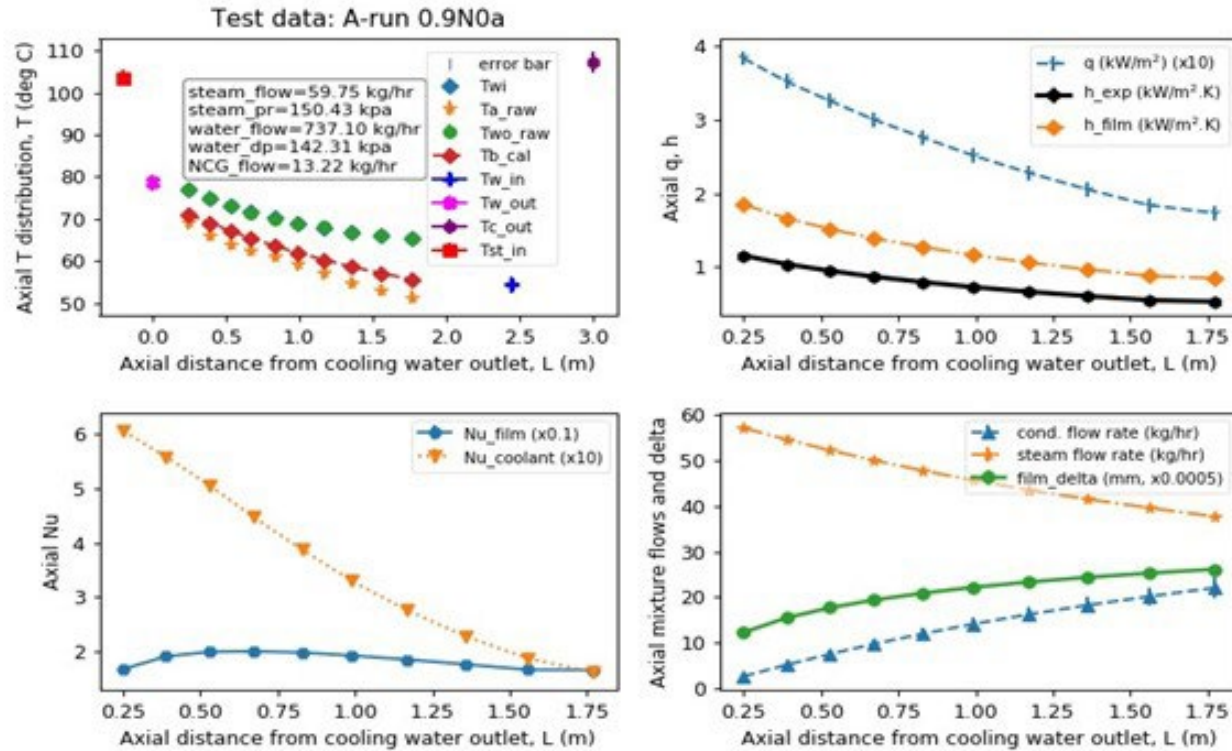


Fig. 5: Test data: A-run0.9N0a (4-in. test section;
NCG: N2, high flow)

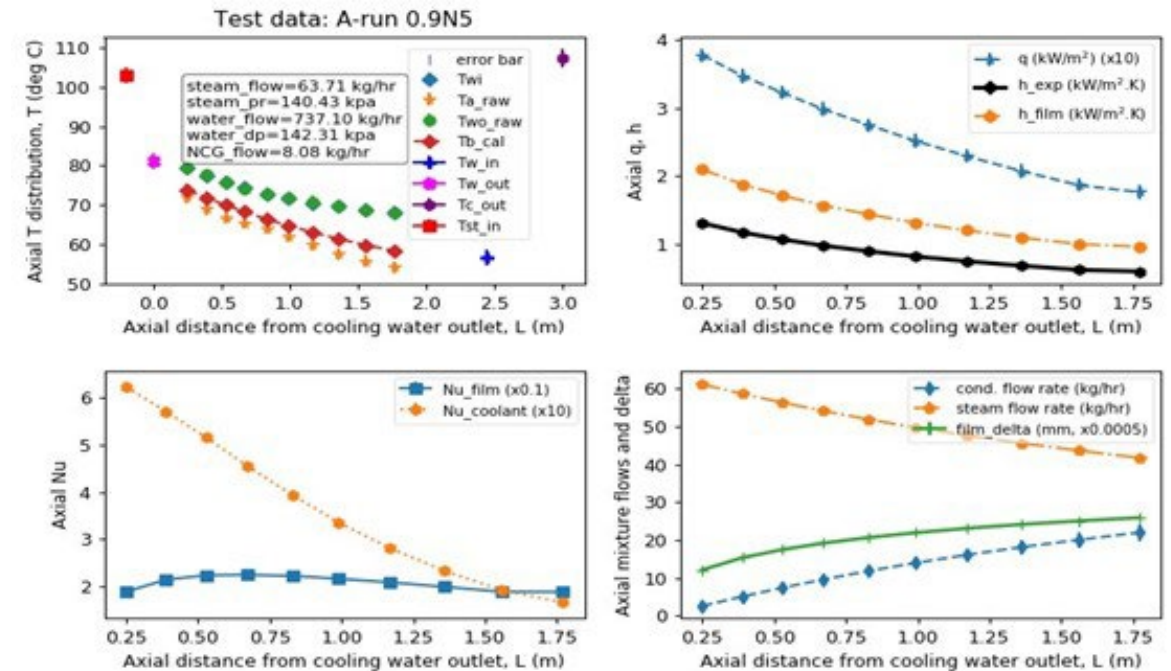


Fig. 6: Test data: A-run0.9N5 (4-in. test section;
NCG: N2, medium flow).

Results and Discussion

The following observations resulted from the experiments and test data analysis:

- Test data showed the CHT, HTC, and condensation rate all decreased with an increase in NCGs. In contrast, these values increased as the steam mass flow increased.
 - The test data were collected at a certain distance from the inlet and outlet to avoid entrance and exit effects.
- A series of similar tests was conducted for varying steam mass flow rates, pressures, and NCG mass fractions. Test data showed qualitative consistency.
 - However, better consistency of the test data would be achievable by introducing adequate control elements (e.g., control valves) to control the testing conditions (e.g., steam-NCG mix inlet mass flow rates, temperature, and pressure), as well as controlling the cooling and condensate discharge.

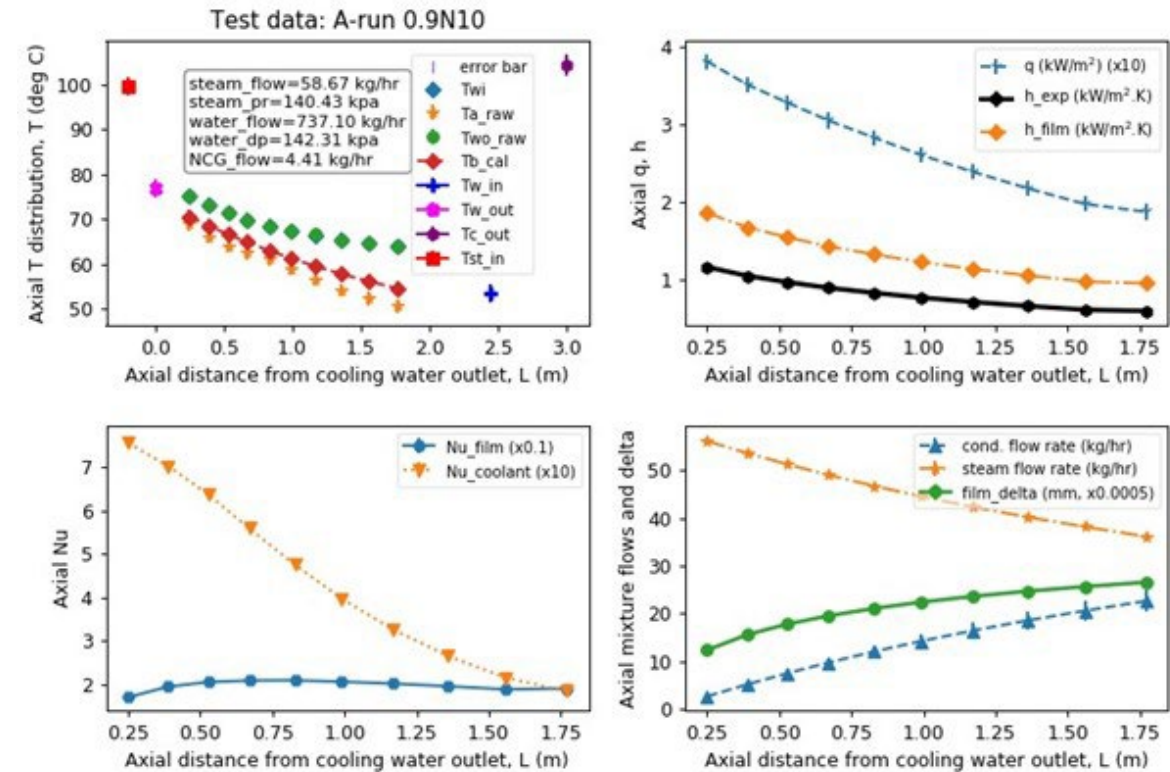


Fig. 7: Test data: A-run0.9N10 (4-in. test section; NCG: N₂, low flow).

Results and Discussion (cont'd)

- The estimated DF, for the various test sections and steam-nitrogen tests exhibited that
 - with an increase in the diameter of the condenser tube, the DF decreased and ranged from 0.1 to 0.7, 0.05 to 0.4, and 0.05 to 0.25 for the 1-in., 2-in., and 4-in. test geometries, respectively.
 - Note that there are a few inconsistent data points for the 2-in. tests, which are outliers.
 - This test dataset could improve verification and validation efforts of computation models and tools [19, 20], which will further improve understanding of the detailed thermal-hydraulics behavior.

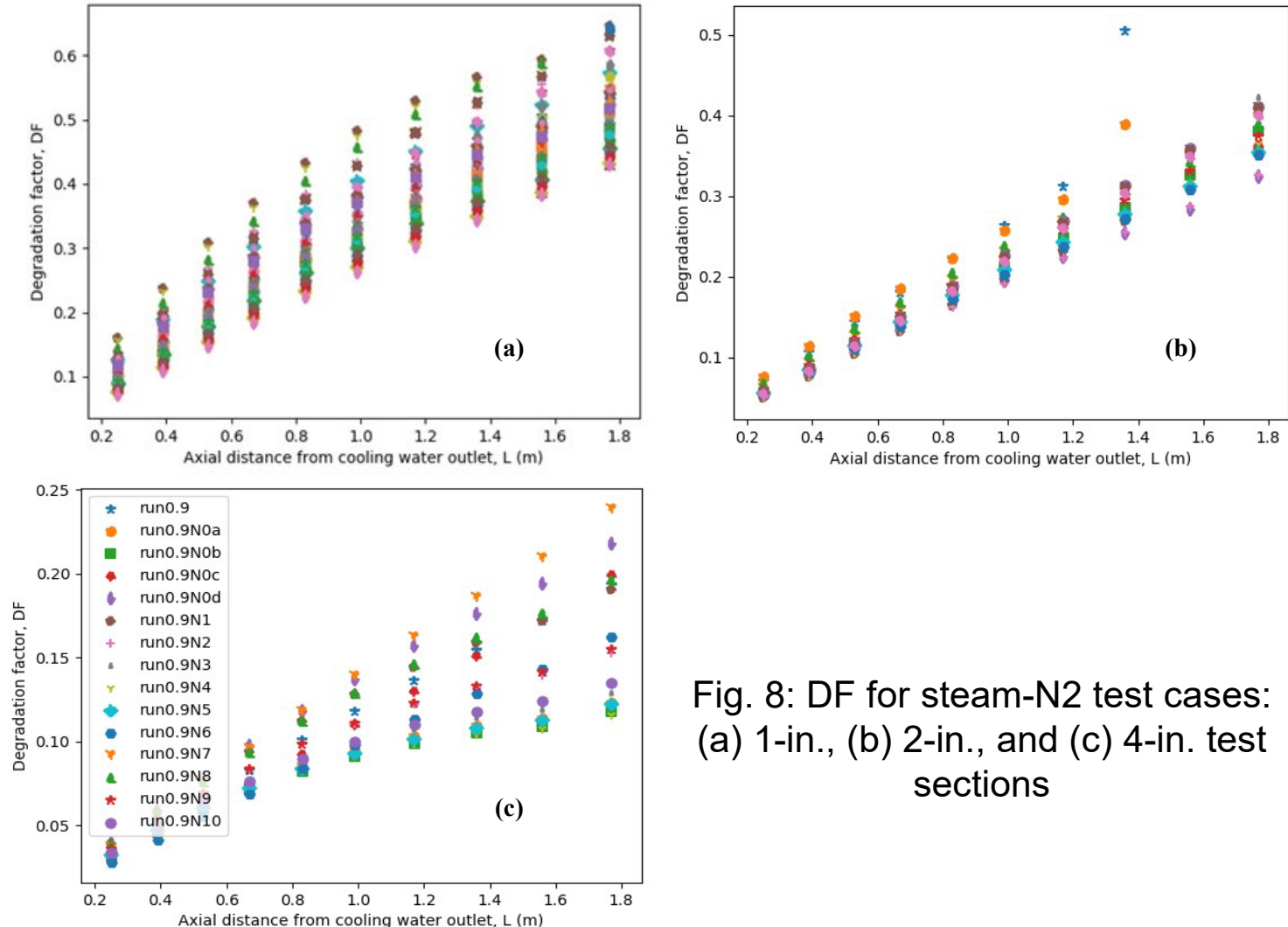


Fig. 8: DF for steam-N₂ test cases: (a) 1-in., (b) 2-in., and (c) 4-in. test sections

Summary, Conclusions, and Path Forward

To analyze the physical phenomena behind SMR PCCSs, this study presents scaled experiments regarding steam condensation in the presence of NCGs. The tests involved vertical downward steam flow and condensation within the inner surfaces of condenser tubes, coupled with annular or pool-cooling methods. A few observations and conclusions can be derived from this study:

- This study focused on later-stage accident phases—mostly long-term cooling—for standard SMRs, which occur
 - about 72 hours into the cooling period following an initiation of an accident and utilized three different scaled geometry test sections.
- Steam condensation considered in the study comprised of film-wise and dropwise; however, for dominating steam condensation in reactor containment during long-term cooling, this study focused on the film-wise type condensation on the containment surface.
- This study presents scaled test data and results for steam condensation in the presence of nitrogen, which simulates air for the containment system. The percentage of air presence in steam release accidents varies over the accident phases.
 - As this study focuses on long-term cooling and film condensation, the physics approximation of NCGs is considered as a barrier to bulk steam for film heat transfer, which is related to NCG partial pressure—an important phenomenon of interest.
- Simple semi-empirical representation of the test data using the degradation factor provides a general relation with experimental and theoretical HTC estimation, including the effect of NCG.
- Future research could encompass validating steam condensation CFD and system code models, and obtaining qualified data, which would identify
 - scaling factors, scaling distortion, and uncertainty associated with geometric and testing conditions, for decision-making to develop qualified test facilities at prototypic conditions.

References

1. IAEA, 2022. “Small Modular Reactors: A new nuclear energy paradigm, pre-print,” International Atomic Energy Agency (IAEA), Vienna, Austria (2022).
2. Schlegel, J. P., and P. K. Bhowmik, 2024. “Chapter 14 - Small modular reactors,” In: Editor(s): Jun Wang, Sola Talabi, and Sama Bilbao y Leon, *Nuclear Power Reactor Designs*, Academic Press, San Diego, CA, USA. Pages 283–308. <https://doi.org/10.1016/B978-0-323-99880-2.00014-X>.
3. U.S. NRC Regulatory Guide 1.203. 2005 [cited 8 January 2024]. Available from: <https://www.nrc.gov/docs/ML0535/ML053500170.pdf> (accessed 2 February 2024).
4. Bhowmik, P. K., C. E. E. Perez, J. D. Fishler, S. A. B. Prieto, I. D. Reichow, J. T. Johnson, P. Sabharwall, and J. E. O'Brien, 2023. “Integral and separate effects test facilities to support water cooled small modular reactors: A review.” *Prog. Nucl. Energy*, 160, 104697. <https://doi.org/10.1016/j.pnucene.2023.104697>.
5. Yadav, M. K., S. Khandekar, and P. K. Sharma, 2016. “An integrated approach to steam condensation studies inside reactor containments: A review,” *Nucl. Eng. Des.*, 300, 181–209. <https://doi.org/10.1016/j.nucengdes.2016.01.004>.
6. Bhowmik, P. K., J. P. Schlegel, and S. Revankar, 2022. “State-of-the-art and review of condensation heat transfer for small modular reactor passive safety: Experimental studies,” *Int. J. Heat Mass Transf.*, 192, 122936. <https://doi.org/10.1016/j.ijheatmasstransfer.2022.122936>.
7. Bhowmik, P. K., J. P. Schlegel, and S. Revankar, 2023. “State-of-the-art and review of condensation heat transfer for small modular reactor passive safety: Computational studies,” *Nucl. Eng. Des.*, 410, 112366. <https://doi.org/10.1016/j.ijheatmasstransfer.2022.122936>.
8. Bhowmik, P. K., and J. P. Schlegel, 2023. “Multicomponent gas mixture parametric CFD study of condensation heat transfer in small modular reactor system safety,” *Exp. Comput. Multiph. Flow*, 2023. 5(1), 15–28. <https://doi.org/10.1007/s42757-022-0136-8>.
9. Lee, K. Y., 2007. “The Effects of Noncondensable Gas on Steam Condensation in a Vertical Tube of Passive Residual Heat Removal System,” Ph.D. thesis, Department of Mechanical Engineering, Pohang University of Science and Technology, Pohang, Korea.
10. Solanki, D. K., N. Dhileeban, R. S. Rao, A. K. Deo, P. K. Baburajan, A. Sridharan, and S. V. Prabhu, 2023. “Steam condensation on a circular tube in the presence of non-condensables (air) in passive containment cooling system,” *Int. J. Heat Mass Transf.*, 213, 124323. <https://doi.org/10.1016/j.ijheatmasstransfer.2023.124323>.

References (cont'd)

11. Bae, B.-U., S. Kim, Y.-S. Park, and K.-H. Kang, 2020. “Experimental investigation on condensation heat transfer for bundle tube heat exchanger of the PCCS (passive containment cooling system),” *Ann. Nucl. Energy*, 139, 107285. <https://doi.org/10.1016/j.anucene.2019.107285>.
12. Haag, M., P. K. Selvam, and S. Leyer, 2020. “Effect of condenser tube inclination on the flow dynamics and instabilities in a passive containment cooling system (PCCS) for nuclear safety,” *Nucl. Eng. Des.*, 367, 110780. <https://doi.org/10.1016/j.nucengdes.2020.110780>.
13. Chen, R., P. Zhang, P. Ma, B. Tan, Z. Wang, D. Zhang, w. Tian, S. Qiu, and G. H. Su, 2020. “Experimental investigation of steam-air condensation on containment vessel,” *Ann. Nucl. Energy*, 136, 107030. <https://doi.org/10.1016/j.anucene.2019.107030>.
14. Bhowmik, P. K., S. Usman, and J. P. Schlegel, 2023. “Film condensation with high heat fluxes and scaled experiments using pure steam for reactor containment cooling,” *Appl. Therm. Eng.*, 229, 120610. <https://doi.org/10.1016/j.applthermaleng.2023.120610>.
15. Vierow, K. M., 1990. “Behavior of steam-air systems condensing in cocurrent vertical downflow,” MS Thesis, University of California—Berkeley, Berkeley, CA, USA.
16. Kuhn, S., P. Peterson, and V. Schrock, 1996. “Determination of the local heat flux in condensation experiments,” *Exp. Heat Transf.*, 9(2), 149–163. <https://doi.org/10.1080/08916159608946519>.
17. Park, H. S., and H. C. No, 1999. “A condensation experiment in the presence of noncondensables in a vertical tube of a passive containment cooling system and its assessment with RELAP5/MOD3.2,” *Nucl. Technol.*, 127(2), 160–169. <https://doi.org/10.13182/NT99-A2992>.
18. Lee, K. Y., and M. H. Kim, 2008. “Experimental and empirical study of steam condensation heat transfer with a noncondensable gas in a small-diameter vertical tube,” *Nucl. Eng. Des.*, 238(1), 207–216. <https://doi.org/10.1016/j.nucengdes.2007.07.001>.
19. Bhowmik, P.K., Schlegel, J.P., Kalra, V. Alam, S. B., Hong, S., & Usman, S. (2022). CFD validation of condensation heat transfer in scaled-down small modular reactor applications, Part 2: Steam and non-condensable gas. *Exp. Comput. Multiph. Flow* 4, 424–434 (2022). <https://doi.org/10.1007/s42757-021-0113-7>.
20. Bhowmik, P.K., Schlegel, J.P., Kalra, V. Alam, S. B., Hong, S., & Usman, S. (2022). CFD validation of condensation heat transfer in scaled-down small modular reactor applications, Part 2: Steam and non-condensable gas. *Exp. Comput. Multiph. Flow* 4, 424–434 (2022). <https://doi.org/10.1007/s42757-021-0113-7>.

Acknowledgment

- The authors thank the Small Modular Reactor Research and Education Consortium and Department of Mining and Nuclear Engineering at Missouri University of Science and Technology for the research support; and Thermal Fluid Systems Methods and Analysis Department at Idaho National Laboratory (INL) for professional development and support.
- **Conflicts of Interest:** The authors declare no conflict of interest.
- **Disclaimer/Publisher's Note:** This information as prepared as an account of work sponsored by an agency of the U.S. government. Neither the U.S. government nor any agency thereof, nor any of their employees, makes any warranty, expressed or implied, or assumes legal liability or responsibility for the accuracy, completeness, or usefulness, of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. References herein to any specific commercial product, process, or service by trade name, trademark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring of the U.S. government or any agency thereof. The views and opinions of authors expressed herein do not necessarily reflect those of the U.S. government or any agency thereof.

Thank you for your attention!