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Standardized Verification of the Cyclus Fuel Cycle Simulator

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Abstract

Many nuclear fuel cycle system modeling codes can analyze transition from a once-through to an advanced nuclear fuel cycle. Verification studies compare various fuel cycle analysis tools to test agreement and identify sources of difference. This paper benchmarks CYCLUS, the agent-based, open-source fuel cycle simulation code, against a verification study [1] for DYMOND [2], VISION [3], ORION [4], and MARKAL [5]. This study reveals that CYCLUS can match the results from other codes closely, with minor differences caused by reactor module behavior.

1. Introduction

Fuel cycle simulators guide and inform Nuclear Fuel Cycle (NFC) research directions and policy choices. Various institutions have developed fuel cycle simulators targeted at their unique needs, using different methods and different structures to simulate the material flow in the nuclear fuel cycle. Algorithmic differences make validation studies necessary to establish confidence in software capabilities and agreement among analysis tools.

A previous validation study [1] compared four well-known NFC simulation codes DYMOND [2], VISION [3], ORION [4], and MARKAL [5]. The results from each code were compared to a set of ‘model solutions’ that were generated from an excel worksheet for different metrics (e.g. fuel loading in reactor, Used Nuclear Fuel (UNF) inventory) in a transition scenario, and showed excellent agreement. We take the input parameters from the validation study [?]

This study benchmarks CYCLUS’ results against that of other well-known codes, such as DYMOND [2], VISION [3], ORION [4], and MARKAL [5]. We take the input parameters and results from a validation study [1] already done for the mentioned tools for a transition scenario from an open fuel cycle to an advanced fuel cycle with reprocessing. In the benchmark [1], the ‘model solutions’ generated from an excel worksheet are compared to each code results, and the results show excellent agreement.

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1.1. CYCLUS

CYCLUS is an agent-based fuel cycle simulation framework [6], which means that each reactor, reprocessing plant, fuel fabrication plant, and other fuel cycle facility is modeled as a discrete entity. A CYCLUS simulation contains prototypes, fuel cycle facilities with pre-defined parameters, that are deployed as `facility` agents. `institution` and `region` agents manage the `facility` agents. A `region` agent holds a set of `institutions`. An `institution` agent can deploy or decommission `facility` agents. Several versions of `Institution` and `region` exist, varying in complexity and functions [7]. `DeployInst` is used as the institution archetype for this work, where the institution deploys agents at user-defined timesteps.

2. Methodology

Feng et al. comprehensively defines simulation parameters sufficient to reproduce the transition scenario in CYCLUS. In this study, we used the CYCAMORE [6] archetype library to model all fuel cycle facilities. CYCAMORE libraries contain simple fuel cycle facility models.

CYCLUS outputs files in either `.sqlite` or `.h5` format. In this study, we used the `.sqlite` format and analyzed the output file using a python script. After post-processing the output data, we overlap the results with the benchmark’s solutions for comparison. The input file and analysis procedures are all in [zenodo].

The analysis and benchmark is performed iteratively, where the we improve the original result by communicating with the authors of the benchmark. From the original result, the reasons for the differences were analyzed and small edits in the source code were made accordingly. Major differences in the source code are not edited but simply explained in detail as to how they contribute to the difference.

3. Fundamental Modeling Differences in Cyclus

CYCLUS has fundamental code differences from some fuel cycle analysis codes used in the benchmark [1].

CYCLUS has a default timestep of a month. The benchmark is evaluated with annual resolution, so cumulative and annual averages were taken. For example, decommissioning of facilities occurs at the end of a timestep, while building of facilities occurs at the beginning of a timestep.

The CYCAMORE recipe reactor depletes half of its core when decommissioned, whereas the codes in the benchmark [1] deplete all their reactors’ fuel when decommissioned. This causes a major discrepancy for the transuranic elements (TRU) inventory. For this study, we changed the CYCAMORE source code to deplete all its assemblies to the depleted recipe. Also, the CYCAMORE recipe reactor treats each batch (and assembly) as a discrete material, while some codes have continuous fuel discharge. This produces differences in the

results because the batches in the benchmark [1] are in time-averaged values. In this study, the Light Water Reactor (LWR) batch size and cycle time is increased, while decreasing the batch number to keep the core size constant. We simply round up the Sodium-Cooled Fast Reactor (SFR) batch number, while the batch size and cycle time are kept constant. This increases the core size by 1.08%, which is negligible, but will be discussed in the results section. We list the differences in table 1.

Table 1: Difference in Batch number and core size

Category	Benchmark [1]	This study
LWR Batches	4.5	3
LWR Batch size [tHM]	19.91	29.86
LWR Core size [tHM]	89.59	89.59
LWR Cycle time	1 year	1.5 years
SFR Batches	3.96	4
SFR Batch size [tHM]	3.95	3.95
SFR Core size [tHM]	15.63	15.8

Note that all the differences could have been mediated by changing the archetype source codes. However, the only change made was the reactor depletion behavior at decommission due to its large impact. Note that the goal of this study is to show current CYCLUS agreement with other codes and identify differences, not to alter CYCLUS to match the other codes.

4. Results

We represent each CYCLUS result as a solid line, and the benchmark solution as a dotted line for visualization. The results are simply a reproduction of the plots displayed in the benchmark. We obtained the benchmark solutions through personal contact with benchmark author Bo Feng at Argonne National Laboratory.

Figure 1 shows the deployed reactor capacity, and figure 2 shows the LWR retirement and SFR deployment. The two plots show exact agreement with the benchmark solutions.

Figure 3 shows the annual fuel loading rate. The initial fuel loading for 100 LWR reactors were edited to match the plot in the verification study results. Note the oscillations for the LWR fuel loading are caused by the refueling period being 18-month refuel cycle for all LWR reactors aggregated into 12-month groups. Note also that the total values are equal for both plots.

Although indistinguishable in figure 3, there is a small difference between SFR fuel loading proportional to the core mass difference, as mentioned in the previous section. Figure 4 shows the differences normalized by the core mass differences, overlapped with the SFR deployment. This shows that the differences only occur during deployment due to the difference in core mass.

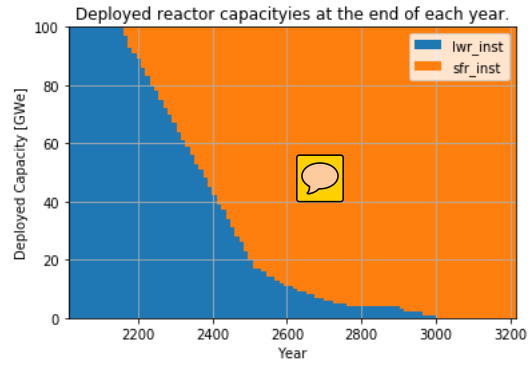


Figure 1: Deployed reactor capacities at the end of each year.

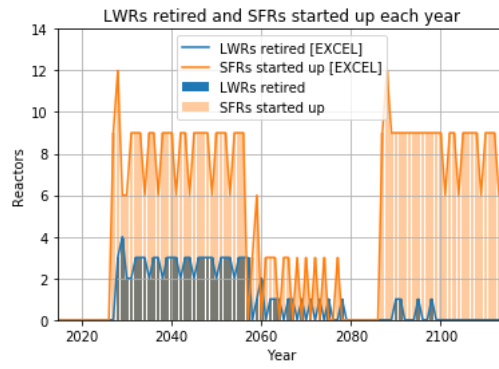


Figure 2: LWRs retired and SFRs started up each year.

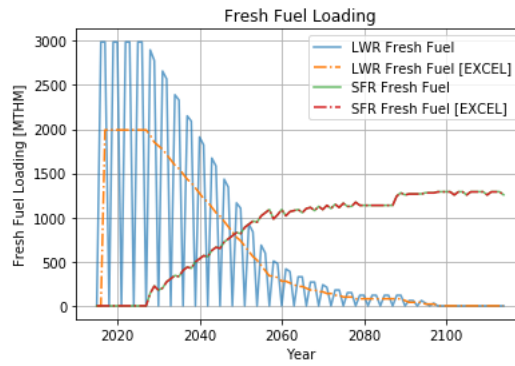


Figure 3: Annual fresh fuel loading rates (first cores and reload fuel).

Figure 5 shows the inventory of discharged UNF in the mandatory cooling stage (four years for LWR, one year for SFR). It also oscillates between the benchmark's solution and converges, caused by the influx and the outflux of UNF into and out of the storage facility. The SFR inventory and fuel loading exactly matches the benchmark solutions, minus the small (1.07%) difference due to core size.

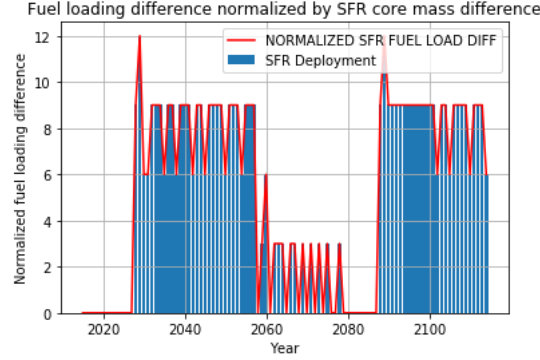


Figure 4: Difference of annual fresh SFR fuel loading rates (Cyclus - Benchmark) normalized by the core mass difference of an SFR due to fractional batch size.

Figure 6 shows similar results for the inventory of cooled UNF waiting for reprocessing. Unlike the previous plot, however, the oscillation peaks meet with the benchmark solution. This is because the cooled UNF inventory is calculated by the cumulative sum of UNF that has been cooled, subtracted by the quantity of UNF reprocessed at that timestep. Thus, the peaks in the oscillation correspond to the cooled inventory in the storage facility before it sends its inventory to reprocessing.

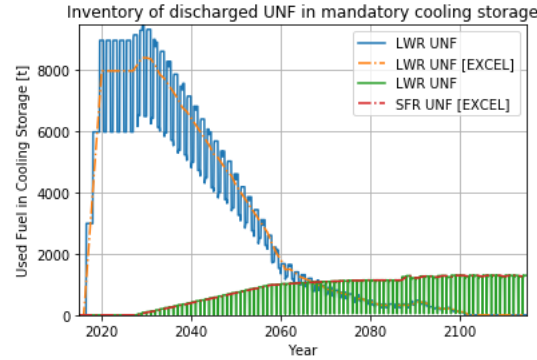


Figure 5: Inventory of discharged UNF in mandatory cooling storage.

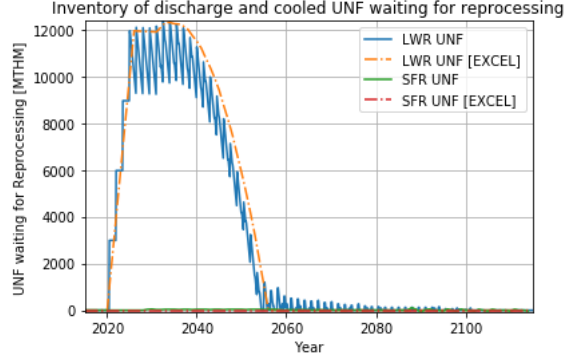


Figure 6: Inventory of discharged and cooled UNF waiting for reprocessing.

Figure 7 shows the reprocessing throughput, which also oscillates around the benchmark solution. No oscillation exists in the beginning because the LWR UNF reprocessing plant throughput peaks at 2,000 tons per year.

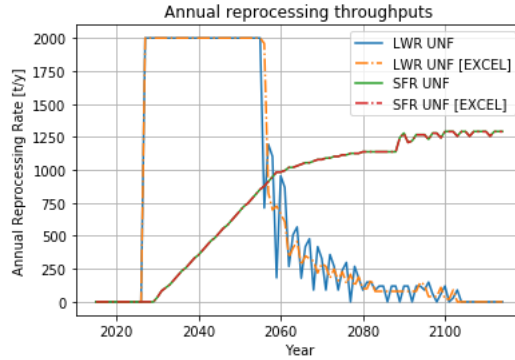


Figure 7: Annual reprocessing throughputs.

Figure 8 shows the inventory of unused TRU recovered from UNF. The CYCLUS results follow the benchmark solutions closely. However, the larger SFR core size causes CYCLUS results to be smaller than the benchmark results, since more TRU is used to start up the newly deployed SFRs. The difference decreases as the SFRs decommission, discharging more UNF (thus TRU) than the benchmark.

5. Discussion

We benchmarked CYCLUS, the agent-based fuel cycle simulator with results from an established verification study and saw good agreement in a transition

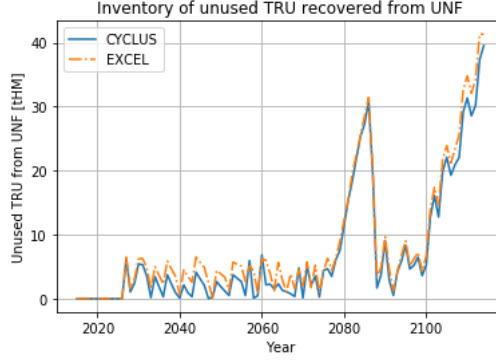


Figure 8: Inventory of unused TRU recovered from UNF.

scenario.

Throughout this work, two major differences were identified that led to the deviation of CYCLUS results to that of the benchmark solution. First, the CYCAMORE reactor depletes only half of its core when decommissioned. Second, CYCLUS, unlike other codes examined in the benchmark (except ORION), fully resolves discrete batches for fuel discharge. We change the first issue by changing one line in the source code.

This study proves CYCLUS as a capable tool for modeling fuel cycle transition scenarios, and shows promise for expansion and future development.

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