



# Sub-sized specimen testing and its relationship with Gen IV Materials Qualification

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*Changing the World's Energy Future*

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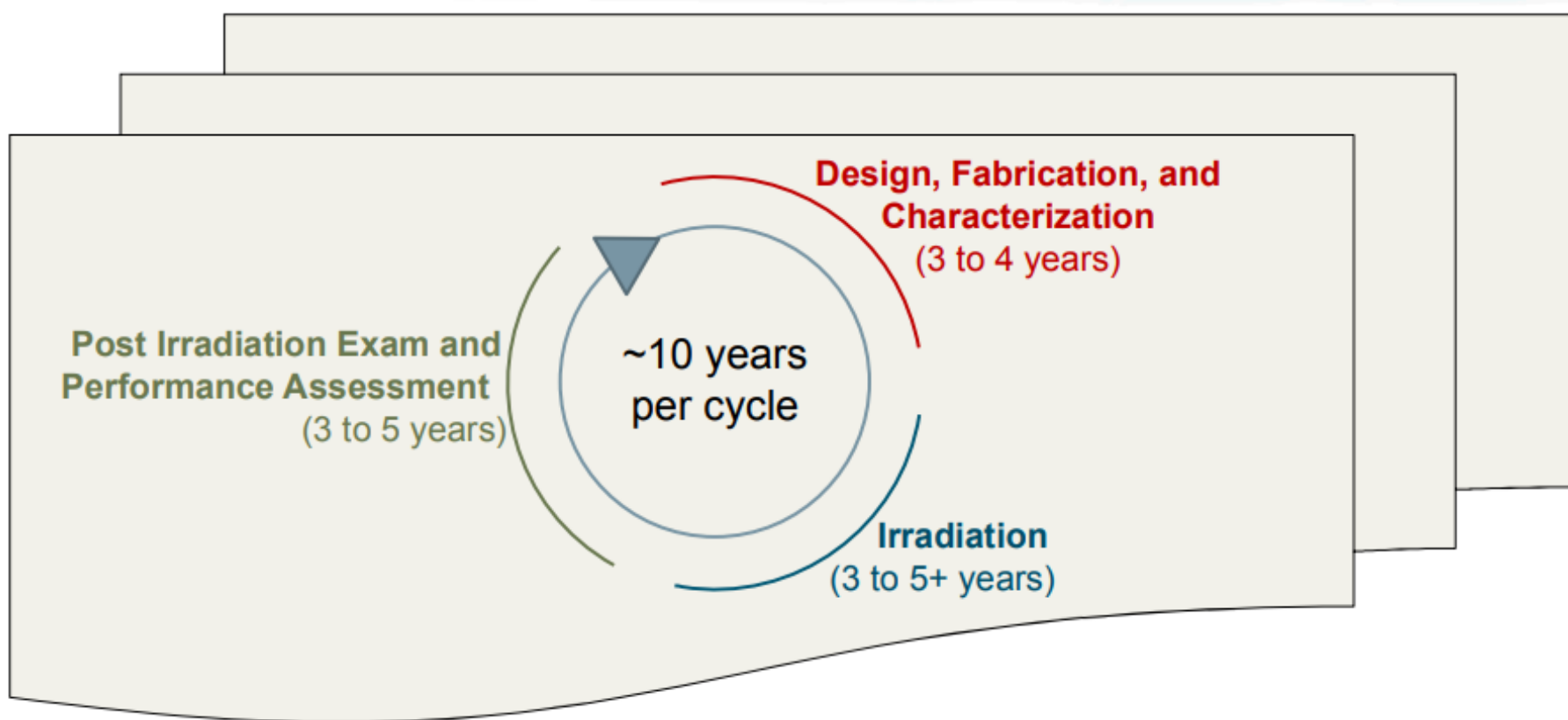
# Sub-sized specimen testing and its relationship with Gen IV Materials Qualification

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## Background

- Qualification of new nuclear structural materials, currently a 20-30 year process, is critical to designing safe, reliable, more efficient Gen IV nuclear reactors that can help the US achieve our clean energy goals.



- INL's Nuclear Materials Discovery and Qualification Initiative (NMDQi) is a program designed to accelerate nuclear materials qualification and reduce that timeline to 3-5 years.
- To achieve this time frame goal, accelerated tests of materials must be performed within test/fast reactors, which are limited in number and have small testing volumes
- This leads to a need to deviate from American Society for Testing and Materials (ASTM)/ American Society of Mechanical Engineers (ASME) standard sized specimens and use sub-sized or miniature specimens.
- The goal of this project was to determine the extent to which sub-sized tensile, creep, and fatigue mechanical testing had been performed on materials applicable to the nuclear industry and the factors necessary to accurately replicate bulk material behavior

## References

Roach, Robert Allen. NMDQi Nuclear Materials Discovery and Qualification Initiative Conference Overview. United States: N. p., 2020. Web.

Džugan, J, Konopik, P, Rund, M, & Prochazka, R. "Determination of Local Tensile and Fatigue Properties With the Use of Sub-Sized Specimens." Proceedings of the ASME 2015 Pressure Vessels and Piping Conference. Volume 1A: Codes and Standards. Boston, Massachusetts, USA. July 19–23, 2015. V01AT01A066. ASME. <https://doi.org/10.1115/PVP2015-4598>

G. E. Lucas, G. R. Odette, M. Sokolov, P. Spa" tig, T. Yamamoto and P. Jung: 'Recent progress in small specimen test technology', J. Nucl. Mater., 2002, 307–311, 1600–1608

T. H. Hyde, W. Sun & J. A. Williams (2007) Requirements for and use of miniature test specimens to provide mechanical and creep properties of materials: a review, International Materials Reviews, 52:4, 213-255, DOI: 10.1179/174328007X160317

## Historical Sub-Sized Testing

### Tensile testing of sub-sized specimens

- Applying the same loading mode (force application methods, loading direction, testing temp. strain rate, etc.) on sub-sized specimens as ASTM standard sized samples leads to observing yield strengths, ultimate tensile strengths, and elongations within +/- 2% of the standard size sample measurement.
- Sub-sized tensile specimens with a minimum width of 6-20 grains within the gauge section were observed to replicate the bulk yield strength, ultimate tensile strength, and elongation within +/- 2%.

### Creep Testing of sub-sized specimens

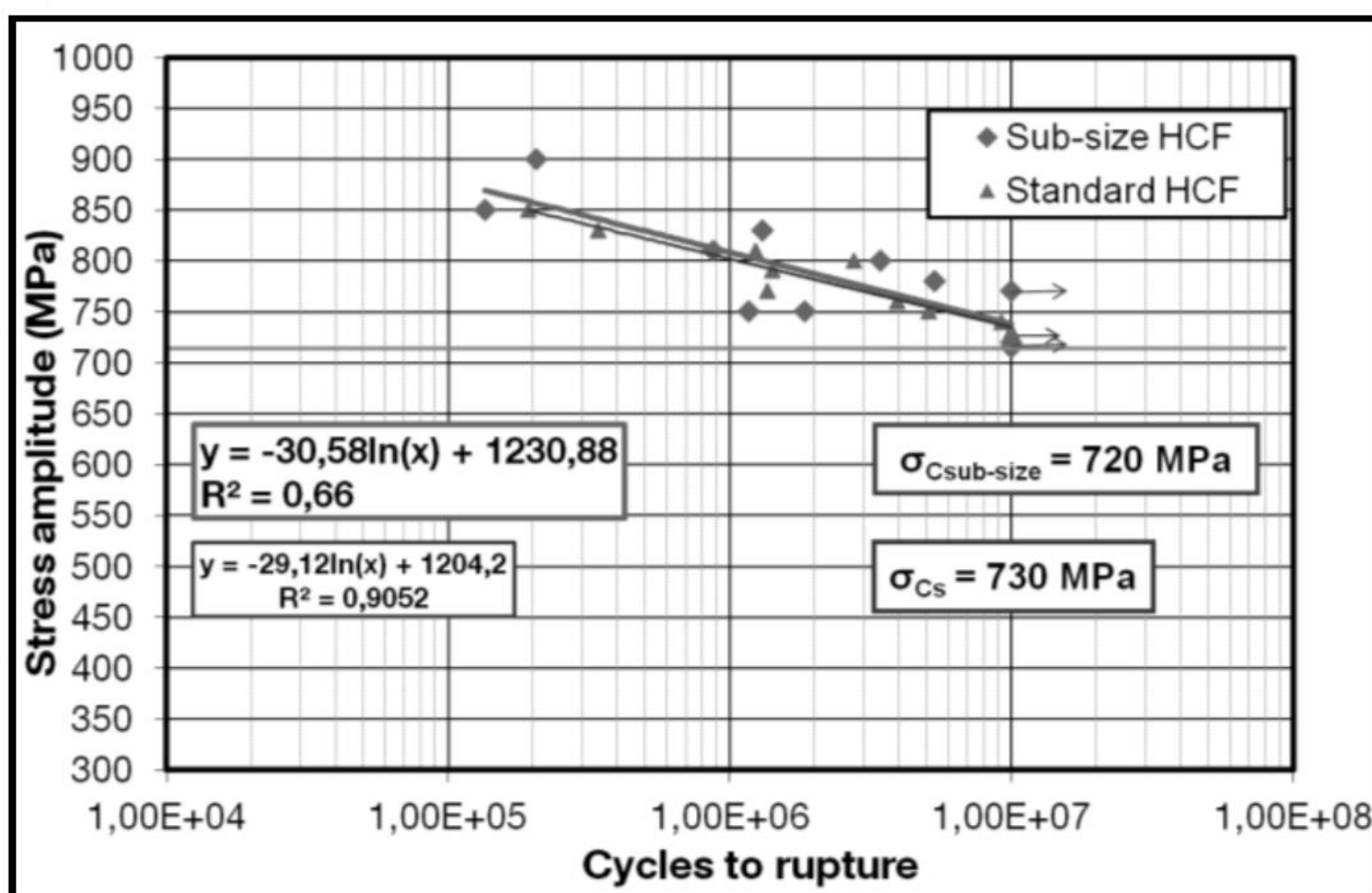
- Uniaxially loaded sub-sized specimen and small punch test (SPT) are common methods of creep testing materials with limited volumes within the nuclear industry.
- SPT however, is limited by the uniqueness of the testing environment within a given lab and the need for correlations specific to the material being tested and specific to the material property of interest, thereby leading to enormous amounts of data generation.

### Fatigue Testing of sub-sized specimens

- Sub-sized fatigue specimens produce fatigue cycle values within +/- 18% and rupture stress values within +/- 7% of ASTM sized samples over the entirety of a Stress-Cycles to failure curve.
- Although scatter exists with sub-sized fatigue specimen data, the graph below shows a linear regression and approximation for the S-N curve is achievable within +/- 1.5% of the fatigue limit.



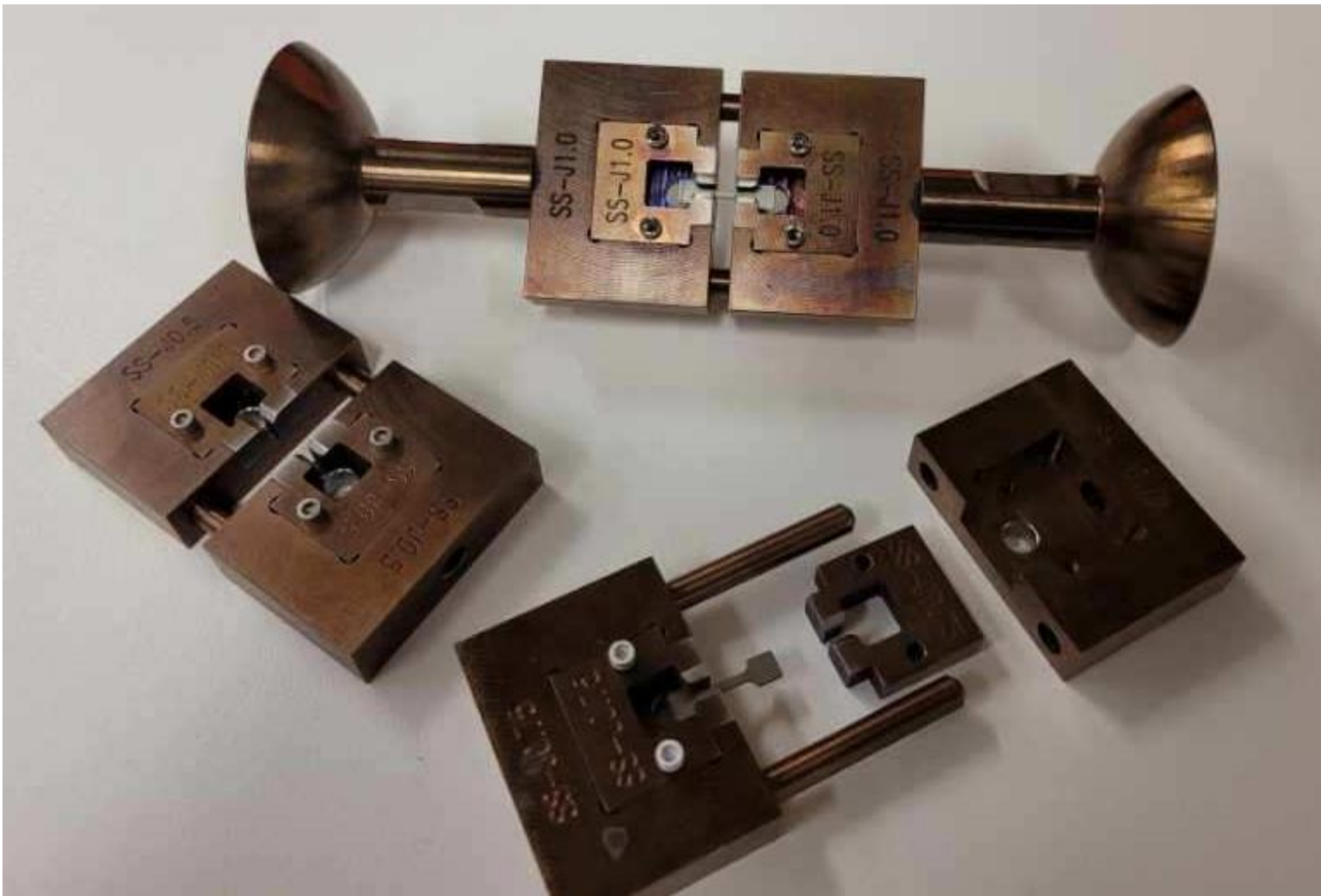
Comparison of standard size and sub-sized fatigue specimens



Results of high cycle fatigue tests performed on standard vs sub-sized specimens

Specimen	Temp	E	YS	UTS	EL
	°C	GPa	MPa	MPa	%
Standard size	20	213.0	1592.0	1648.0	1.0
Micro-Tensile Test	20	213.3	1593.5	1645.6	1.3

Standard vs Micro-Tensile test at 20C data showing Young's modulus (E), Yield Strength (YS), Ultimate Tensile Strength (UTS), and Elongation (EL) within +/- 2%



Clevises for SS-J3 creep testing currently under design and development at INL

## Current & Future Plans

- Given that the literature demonstrates governing factors in sub-sized specimen testing is geometry agnostic, SS-J3 sub-sized specimen have been identified as an ideal sample size due to their limited volume requirements and ubiquity in Nuclear Science User Facilities.
- Development of clevises and mechanical testing systems for testing of irradiated SS-J3 sub-sized specimens has commenced as part of NMDQi.
- INL plans to initially creep test irradiated Inconel 617 SSJ3 sub-sized specimens as copious material testing data of Inconel 617 ASTM standard sized samples exists within INL's previous experimental results.
- After comparing with Inconel 617 data, 709, 800H, and 316H alloys will be tested as these alloys have shown promise for future viability in high temperature/Gen IV reactor structural applications.

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