



# Standardized Irradiation Testing and Multi-Modal Characterization for Advanced Nuclear Materials

May 2022

*Changing the World's Energy Future*

Geoffrey L Beausoleil



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# **AMMT Environmental Effects Testing and Characterization Roadmap for FY22-FY27**



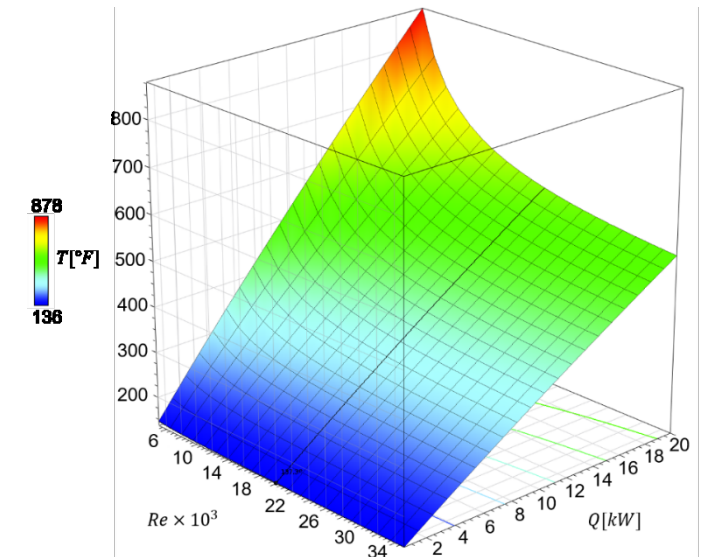
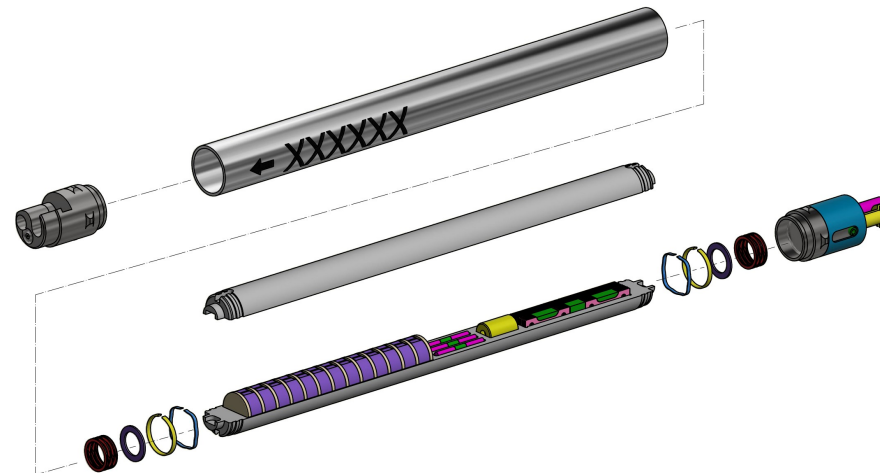
# INL Testing and Characterization Efforts

- Development of standard irradiation capsule ISHA-1
  - Using for SS316H and 16-8-2 Weld Metal
- In-Situ Property Measurement
  - Demonstration of single head thermal reflectance stage within G4 Hydra
  - Development of additional modalities using the feedthrough
- ANCERS, Application of Non-destructive Combinatorial Examination of Radioactive Samples
  - Design of Gamma-Ray Emission Tomography Assay (GRETA) table
  - Developing reconstruction methods using standard samples
- High temperature alloy characterization

# FY21 - Universal Drop-in Capsule for ATR Testing

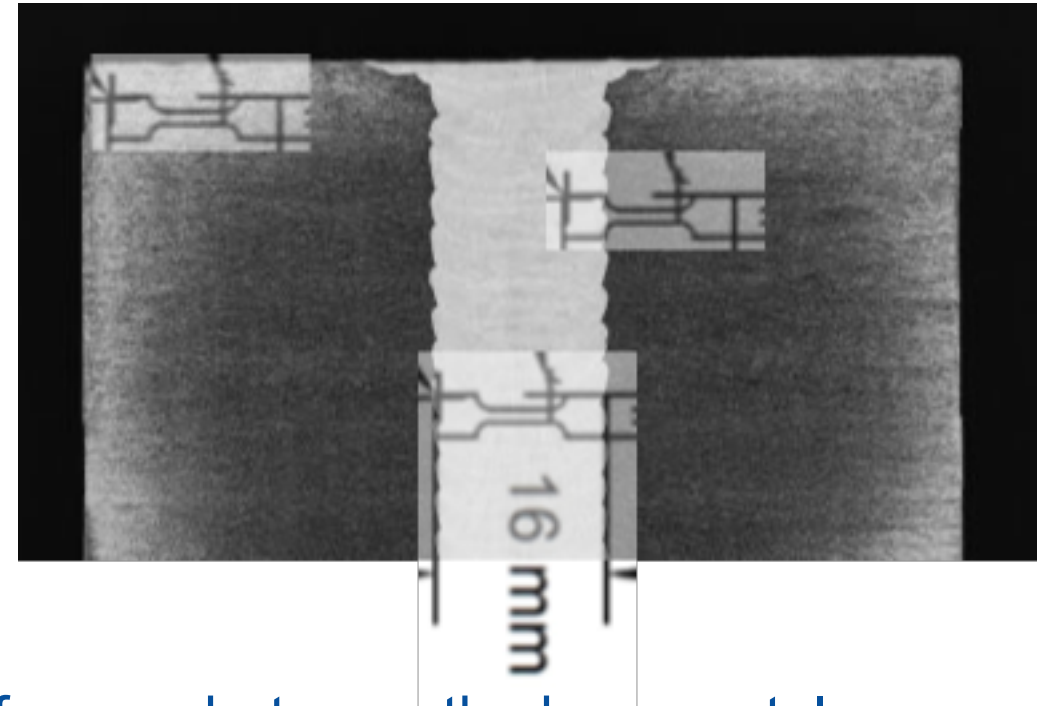
- Under NMDQi, we developed a semi-universal drop-in capsule for irradiation in the Advanced Test Reactor (ATR) called the Irradiation System for High-throughput Acquisition (ISHA-1). This utilizes a standard outer capsule that can facilitate both fuel experiments or structural material specimens in a variety of ATR Positions.
  - Structural material testing can support compact tension, bend bar, and tensile specimen geometries for temperatures up to 800 °C.
  - Utilizes concentric ASME pressure boundaries in order to support more reactive material irradiation testing within ATR, such as molten salts and other reactive materials.

ISHA is a semi-universal drop-in capsule for ATR with an extensive, bounding safety case



# SS316 Irradiation Test

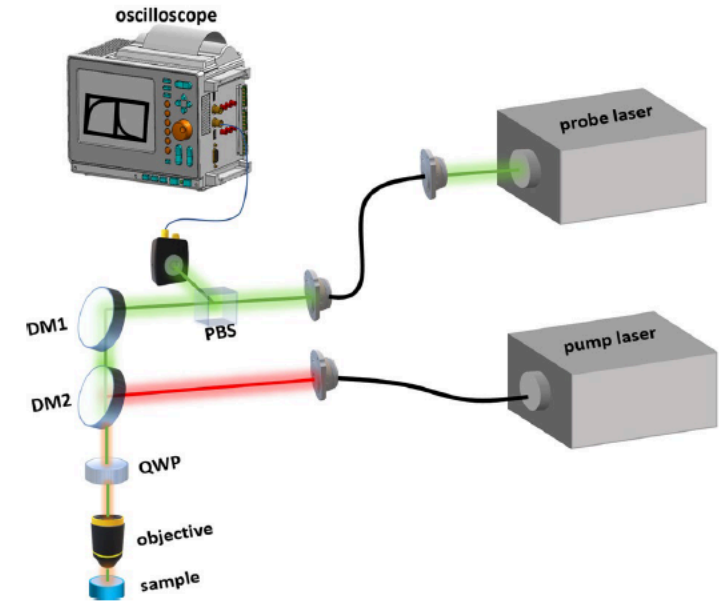
- Partnership with Kairos Power ARDP award and NMDQi efforts
  - Irradiating welded SS316H with AWS 16-8-2 weld metal
  - Temperatures of 550 °C and 650 °C
  - Target doses of <1 dpa, 1-2 dpa, 10-15 dpa
- Objectives will be to mechanically characterize difference between the base metal, weld metal, and heat affected zone
- National Reactor Innovation Center (NRIC) is funding the design, assembly, and deployment of three in-cell load frames to perform the mechanical tests
- Multiple creep rupture lifetimes being investigated
- FY22-FY27 effort
  - Irradiation and PIE funded through ARDP award



High temperature creep testing of 316 and small scale specimens

# Thermal Reflectance Method and the SPTR

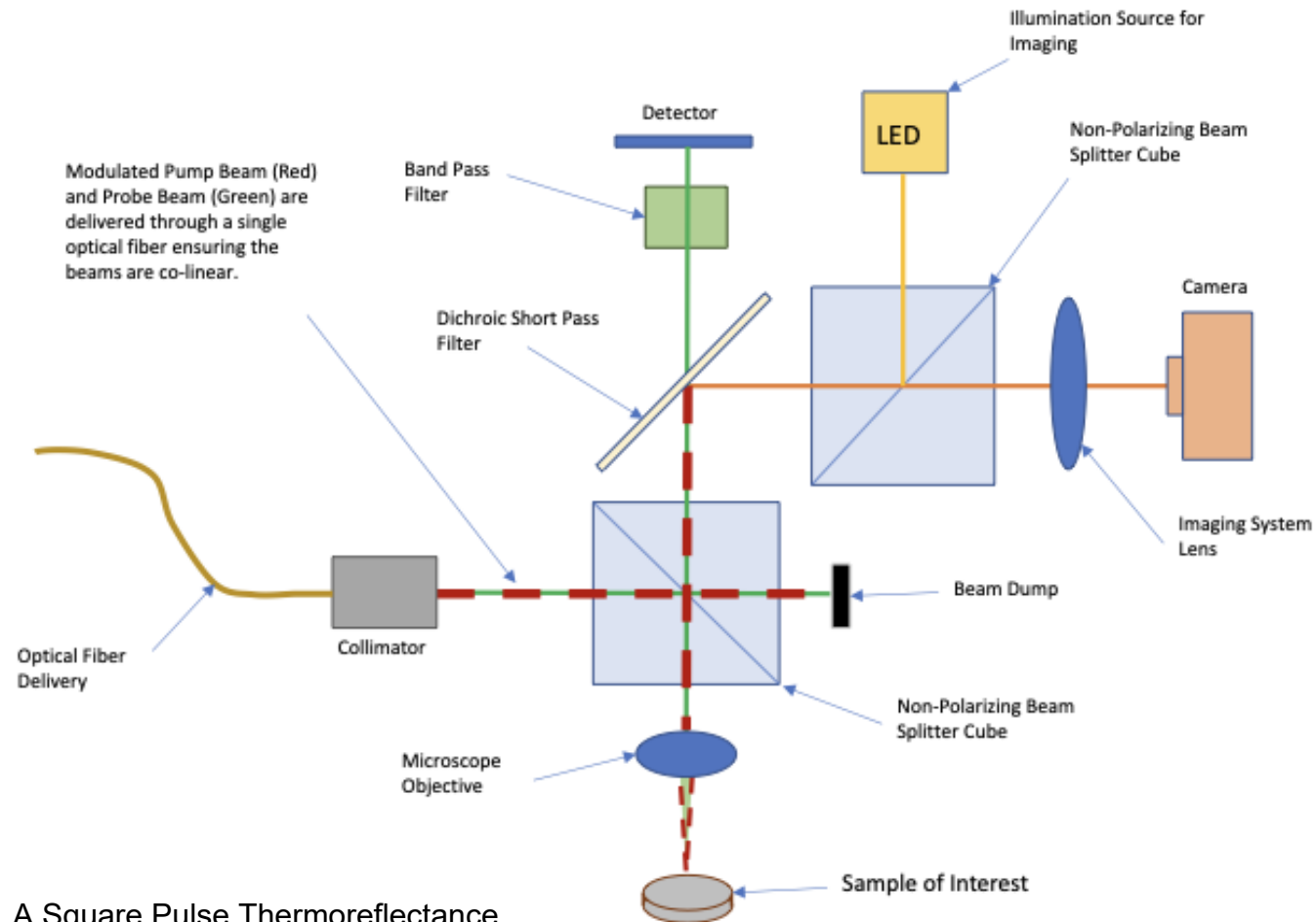
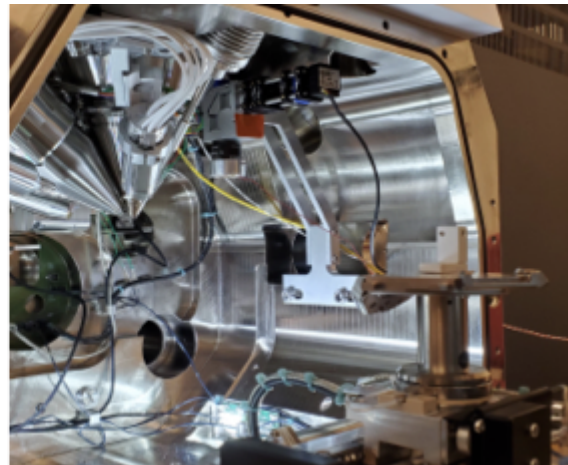
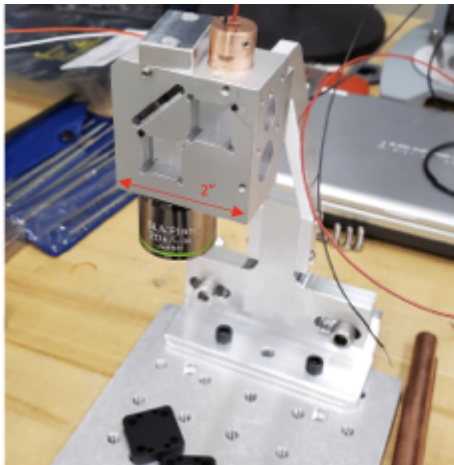
- Thermal reflectance methods utilize the small, local change in optical reflectance of a material based upon its temperature
  - Can compare temperature vs frequency (frequency domain) or temperature vs time (time domain)
  - SPTR utilizes a time domain interpretation
- Thermal reflectance is the basis for the Thermal Conductivity Microscope (TCM) at the Irradiated Materials Characterization Laboratory (IMCL)
- The square pulse thermal reflectance (SPTR) method was developed to support in-situ testing within the G4 Hydra PFIB/SEM
  - Spot sizes on the order of  $2\ \mu\text{m}$  on the specimen surface



**Explicitly correlating  
thermal/mechanical  
properties to  
microstructural features  
during PIE**

# Design of Thermal Reflectance Head in G4 Hydra

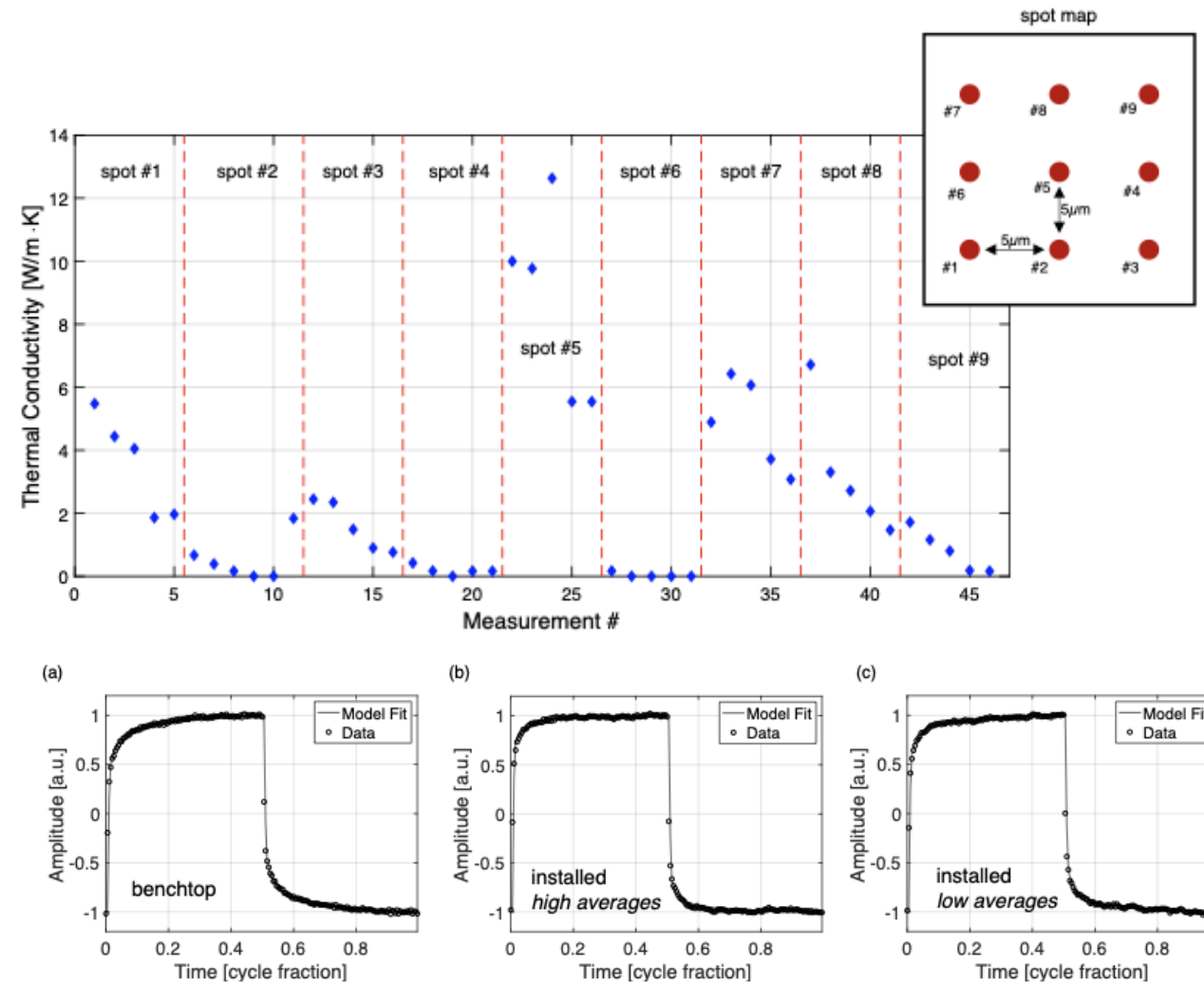
- In-Situ testing property testing during microscopy
- Stage compresses both the probe and pump lasers into a single optic that can support laser change outs from outside the SEM
- Feed through a vacuum port on the SEM chamber



A Square Pulse Thermoreflectance Technique for the Measurement of Thermal Properties  
Y Wang, V Chauhan, Z Hua, R Schley, CA Dennett... - International Journal of Thermophysics, 2022

# Square Pulse Thermal Reflectance (SPTR) Results

- Investigation of the method focused on identifying sensitivities and proving the model
  - Spot size, film (transducer) size, and model variables
- Emphasis was placed on relationships between material conductivity and film thickness
- Validation with BK7 glass and gold films
  - Benchtop was within 3% of known values for BK7 ( $\sim 1$  W/m/K)
  - Installed values were initially not good ( $\sim 3$ -4 W/m/K)



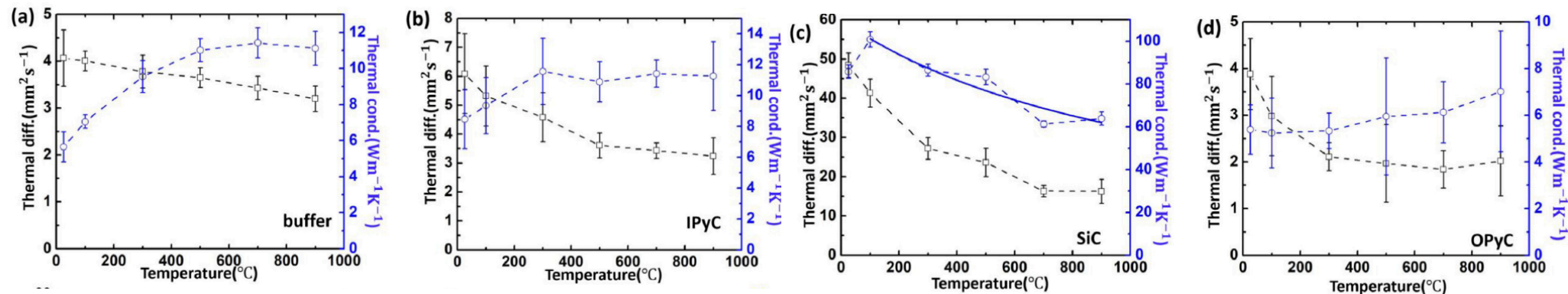
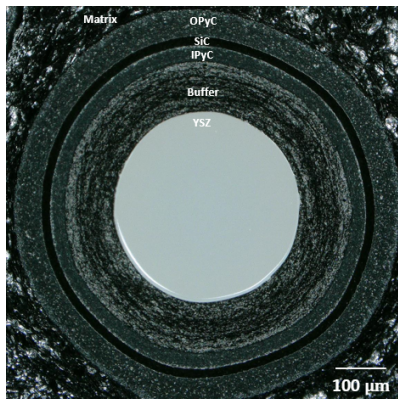


# Investigation of TRISO Particle in G4

- A tri-isotrostructure (TRISO) fuel particle was used as a basis for demonstrating the SPTR instrument within the G4
  - Well established material properties for comparison
  - Small, well defined layers to distinguish the results from
- Results were also compared to a similar instrument, the TCM at IMCL

		SPTR (W/mK)*	TCM	
			Therm cond.(W/mK)	Therm diff.(mm <sup>2</sup> /s)
RT	YSZ	2.22	2.65	0.9
	Buffer	3.64	3.25	3.25
	IPyC	4.75	5.16	5.56
	SiC	74.1	84.3	38.3
	OPyC	4.45	3.96	3.52

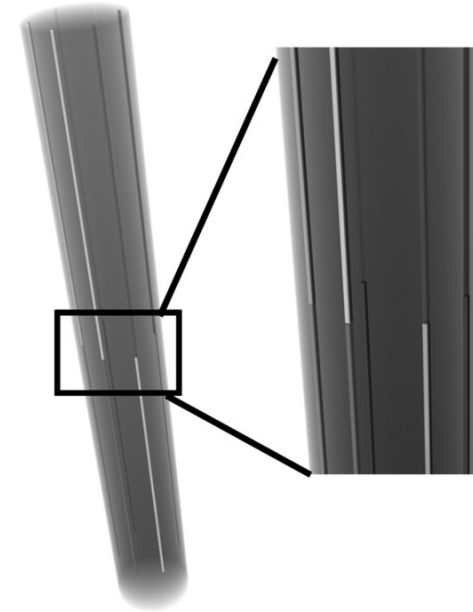
\*assume heat capacity from literature



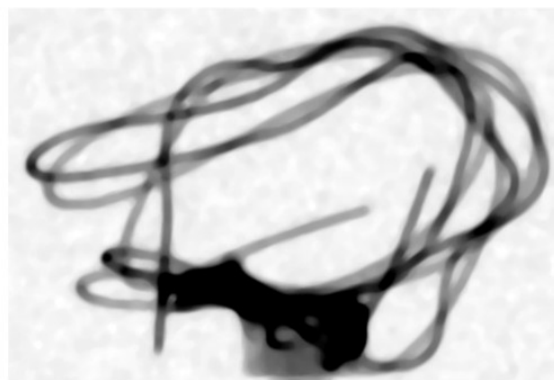
Thermal properties measurement of TRISO particle coatings from room temperature to 900 °C using laser-based thermoreflectance methods  
 Y Wang, Z Hua, R Schley, G Beausoleil II, DH Hurley - Journal of Nuclear Materials, 2022

## FY21: Began Development of Integrated Tomographic Methods for Characterization for Irradiated Materials

- To increase the throughput of materials testing and characterization, combinatorial tomographic methods are under development with the goal of increasing the scientific impact of analysis but combining the data output of a single sample characterized with multiple investigative methods.
  - Application of Non-destructive Combinatorial Examination of Radioactive Samples (ANCERS)
  - Provide isotopic and spatial resolution  $<10\text{ }\mu\text{m}$  on large, dense irradiated specimens



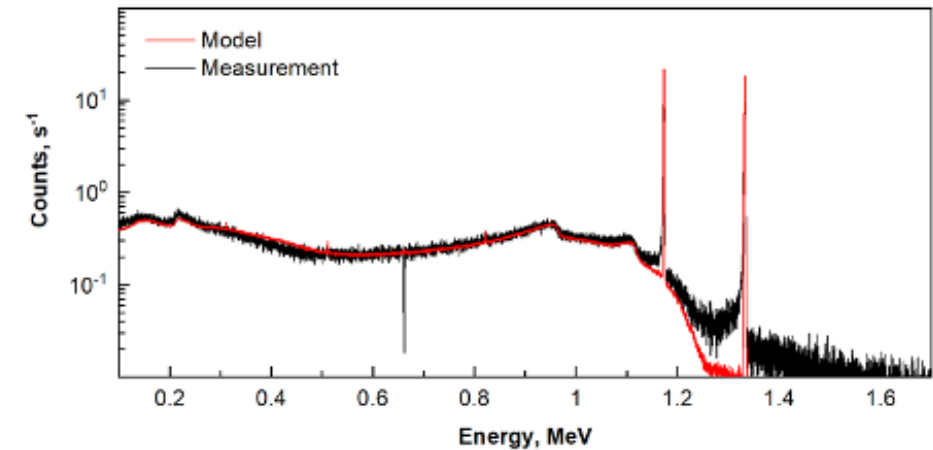
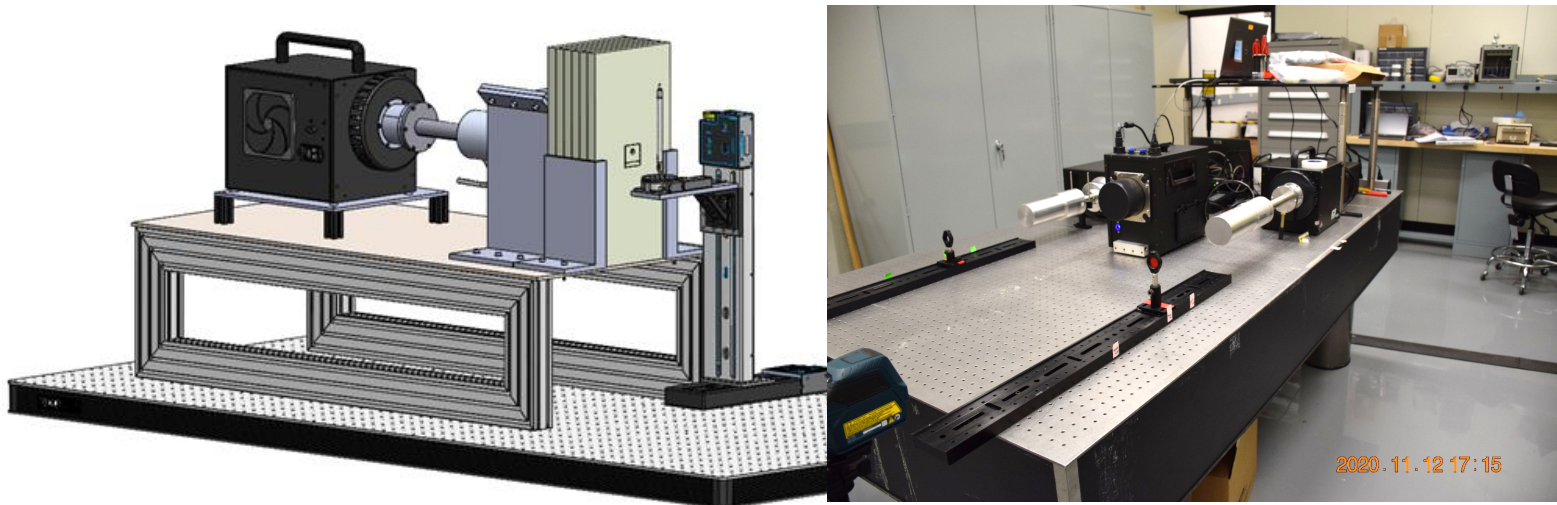
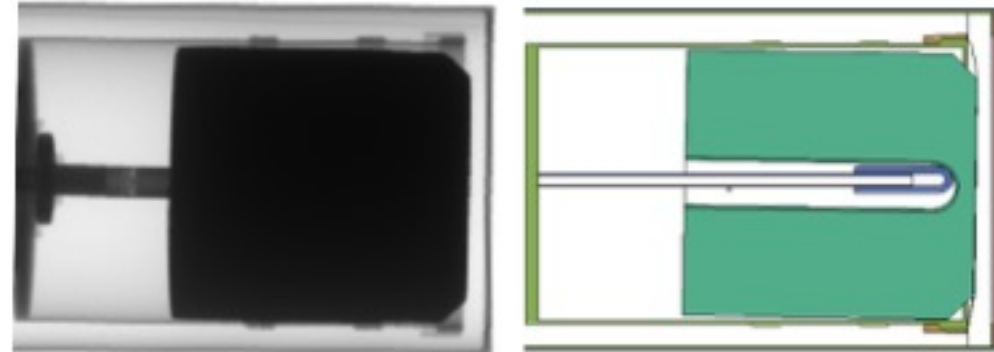
Combining tomographic methods allows each to make up for the others' shortcomings, thus improving the value proposition of tomography on irradiated materials



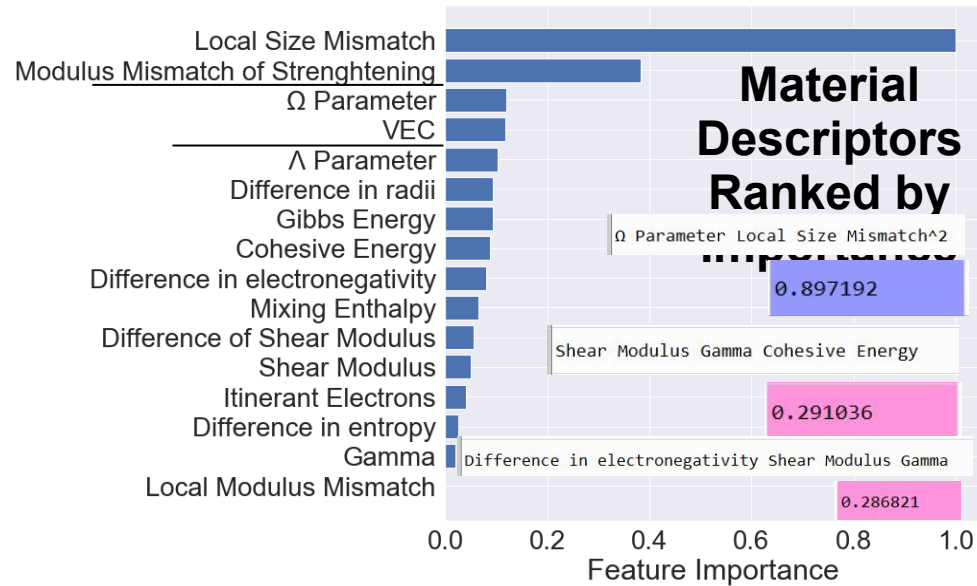


# Gamma Ray Emission Tomography Assay (GRETA) System

- Gamma emission tomography (GET) instrument to support ANCERS
- A high purity Ge detector was received and characterized
- Unfortunately, there were some problems with the received detector and the vendor is in process of replacing it.



# Multi-Principal Element Alloys for Nuclear Applications



## B Group:

- FeCrNiCoCu
- FeCrNiCo
- FeCrNiMnCo

## BB Group:

- FeCrNiMnSiC
- FeCrNiMnSiCMo
- FeCrNiMnAlCoSiCCuTi
- FeCrMn
- FeCrMnCo
- FeCrMn\*

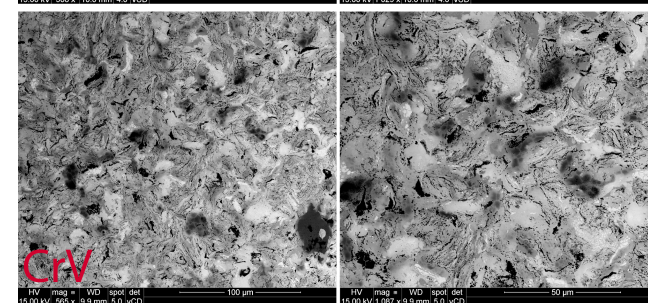
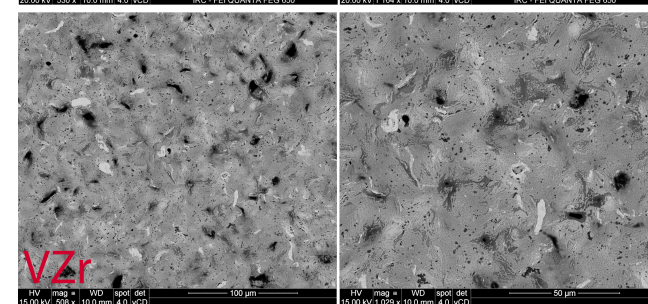
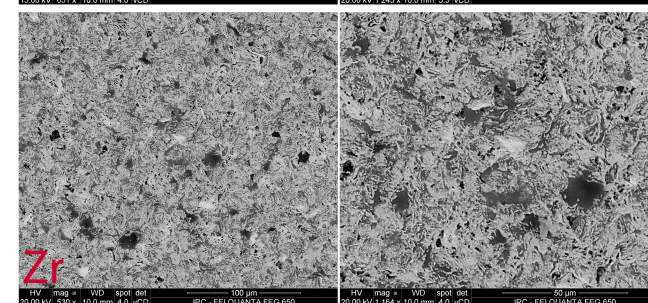
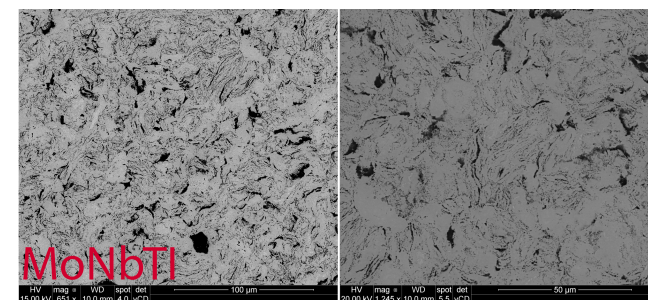
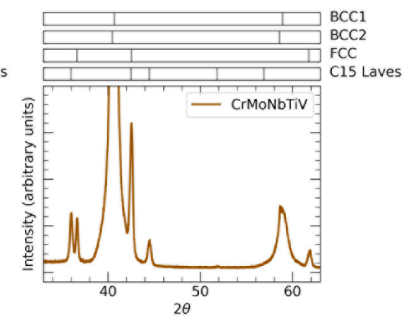
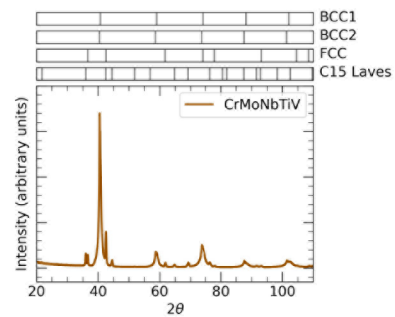
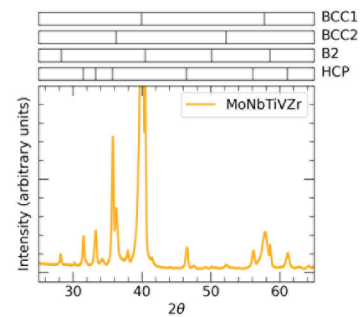
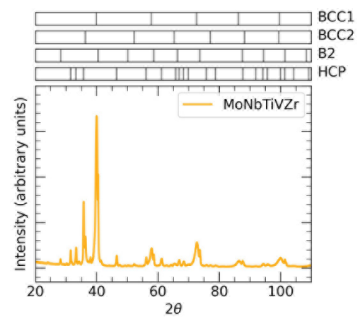
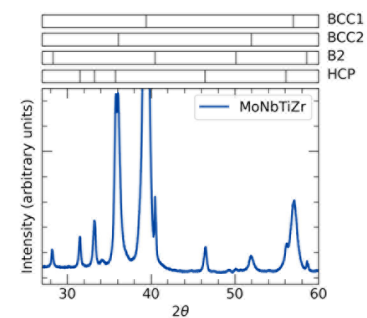
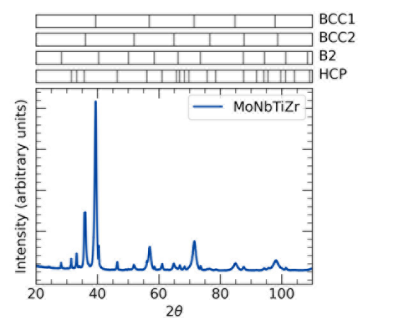
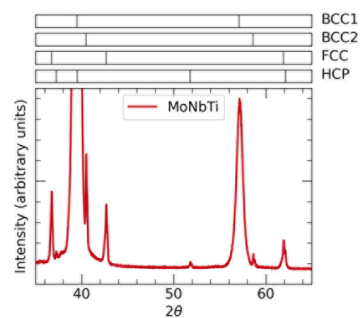
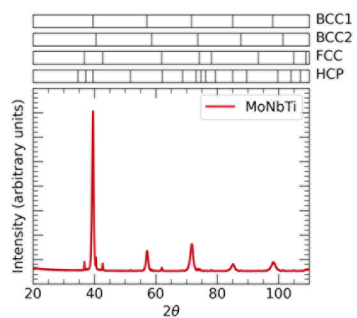
## D Group:

- CrAlZrMoNbTa
- CrAlZrMoNb
- CrAlZrMoNbN
- CrAlZrMoNbNTa<sub>2</sub>
- TiZrVMoNb
- Ti<sub>2</sub>ZrHfV<sub>0.5</sub>Mo<sub>0.2</sub>
- TiZrHfNb
- TiZrHfNbTa
- FeCrVTaW
- CrVTaW

- CrMnVTaW

# ds

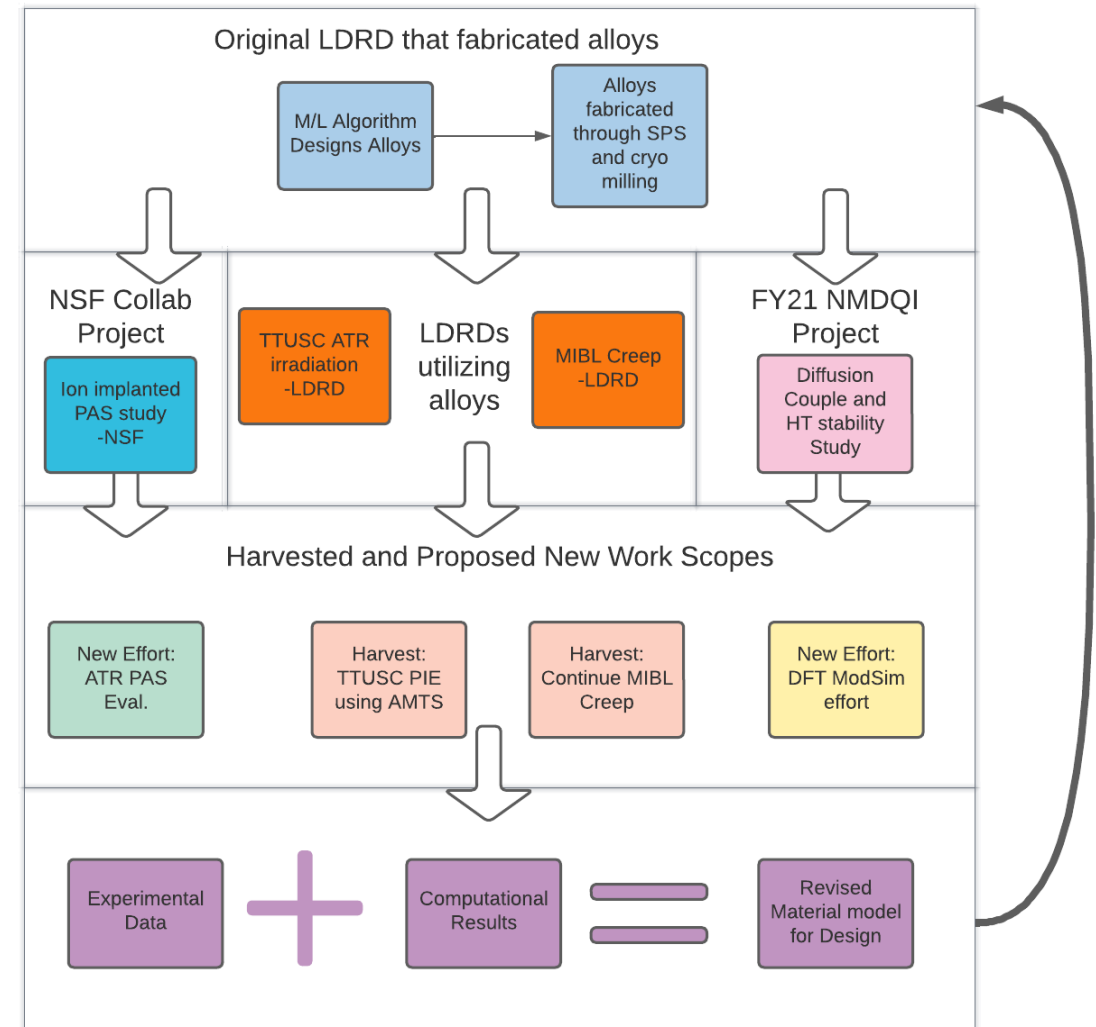
- sdfsd





# FY23-FY25 Harvested LDRDs

- Advanced Alloy Design
  - Machine learning algorithm to predict material mechanical properties
  - Cryogenic milled powders and SPS to fabricate alloys
  - Phase and mechanical testing
- High temperature phase stability and property assessment
- TTUSC – Neutron irradiation and mechanical testing
- MIBL Creep Rig – in-situ ion irradiation and creep testing
- NSF Positron Annihilation Spectrometry
- Addition of ModSim development for atomistic behaviors



Demonstrates  
experimental framework  
for testing new alloys

# Utilizing ATF-2 Irradiation Opportunities

- Vendors often miss insertion dates and are replaced by dummy capsules
  - Conditions are within a flux trap typical of PWR core
  - 300 series SS
  - Simple cylinder for analysis credit
  - Typical minimum irradiation time of 2-3 cycles in the position
- The source of these cylinders is somewhat irrelevant and so the option of fabricating A/M SS316 cylinder
  - Fabricate a collection of A/M SS316 capsules with varying parameters and keep on hand to perform irradiations when available

Essentially a 'free' ATR  
irradiation in PWR Conditions





Idaho National Laboratory

# High Temperature Multi-Principal Element Alloys

