



NT-1 - 5 Results in ATR: PRELIMINARY NUCLEAR TESTING RESULTS AFTER CORE INTERNALS CHANGEOUT #6 IN THE ADVANCED TEST REACTOR

Changing the World's Energy Future

Nathan Manwaring, Kurt E Lombard



DISCLAIMER

This information was prepared as an account of work sponsored by an agency of the U.S. Government. Neither the U.S. Government nor any agency thereof, nor any of their employees, makes any warranty, expressed or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness, of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. References herein to any specific commercial product, process, or service by trade name, trade mark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the U.S. Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the U.S. Government or any agency thereof.

NT-1 - 5 Results in ATR: PRELIMINARY NUCLEAR TESTING RESULTS AFTER CORE INTERNALS CHANGEOUT #6 IN THE ADVANCED TEST REACTOR

Nathan Manwaring, Kurt E Lombard

October 2022

**Idaho National Laboratory
Idaho Falls, Idaho 83415**

<http://www.inl.gov>

**Prepared for the
U.S. Department of Energy
Under DOE Idaho Operations Office
Contract DE-AC07-05ID14517**

October 2022

Nathan Manwaring
Kurt Lombard

Advanced Test Reactor



PRELIMINARY NUCLEAR TESTING RESULTS AFTER CORE INTERNALS CHANGEOUT #6 IN THE ADVANCED TEST REACTOR

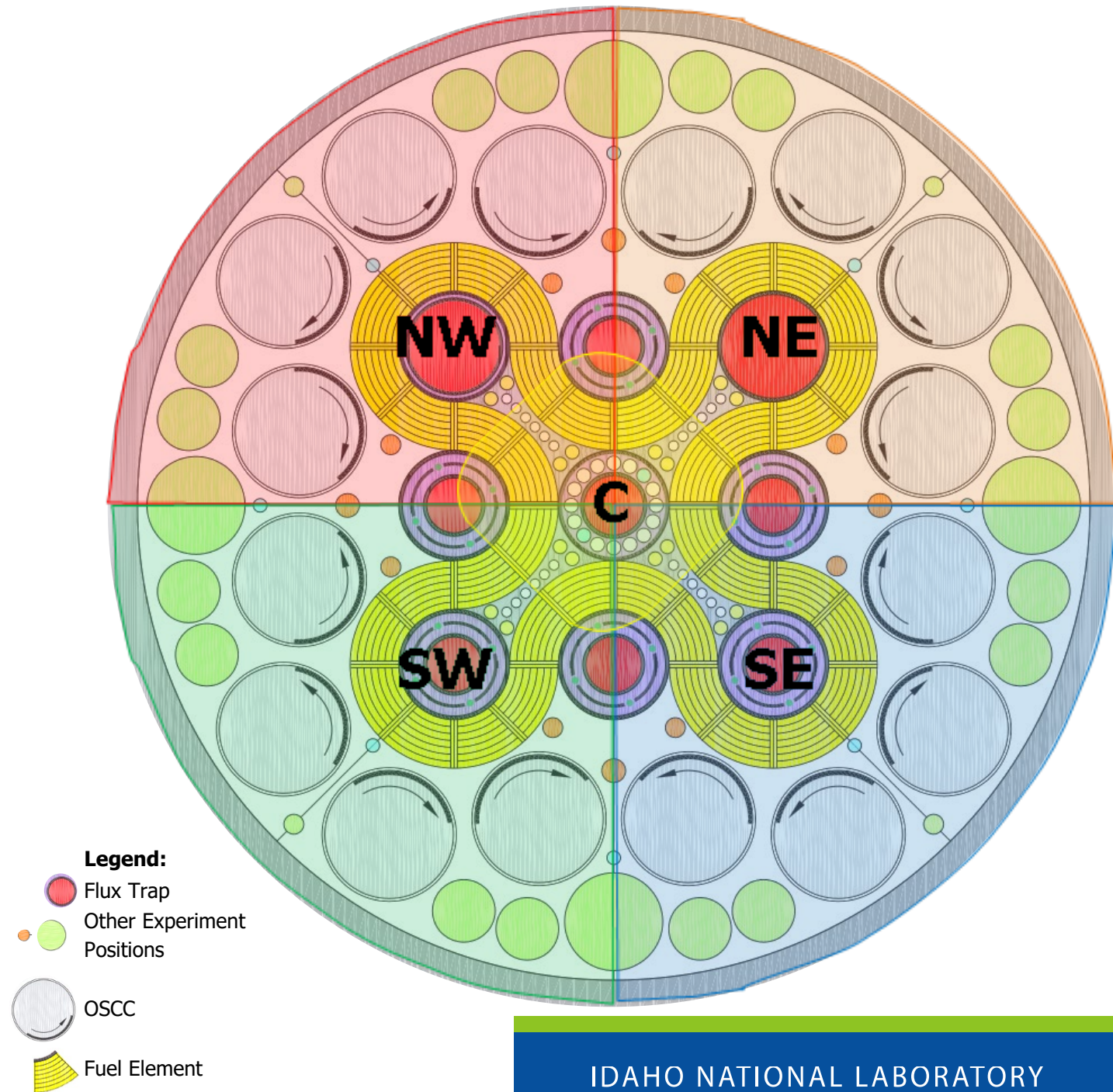
Outline

- Introduction to Advanced Test Reactor (ATR)
 - Idaho National Laboratory
 - Fuel Arrangement
 - Flux Traps
- Low-power Tests
 - NT-1 – 6
 - Results of NT-1 – 5
- Power Escalation Tests
 - NT-7 – 12



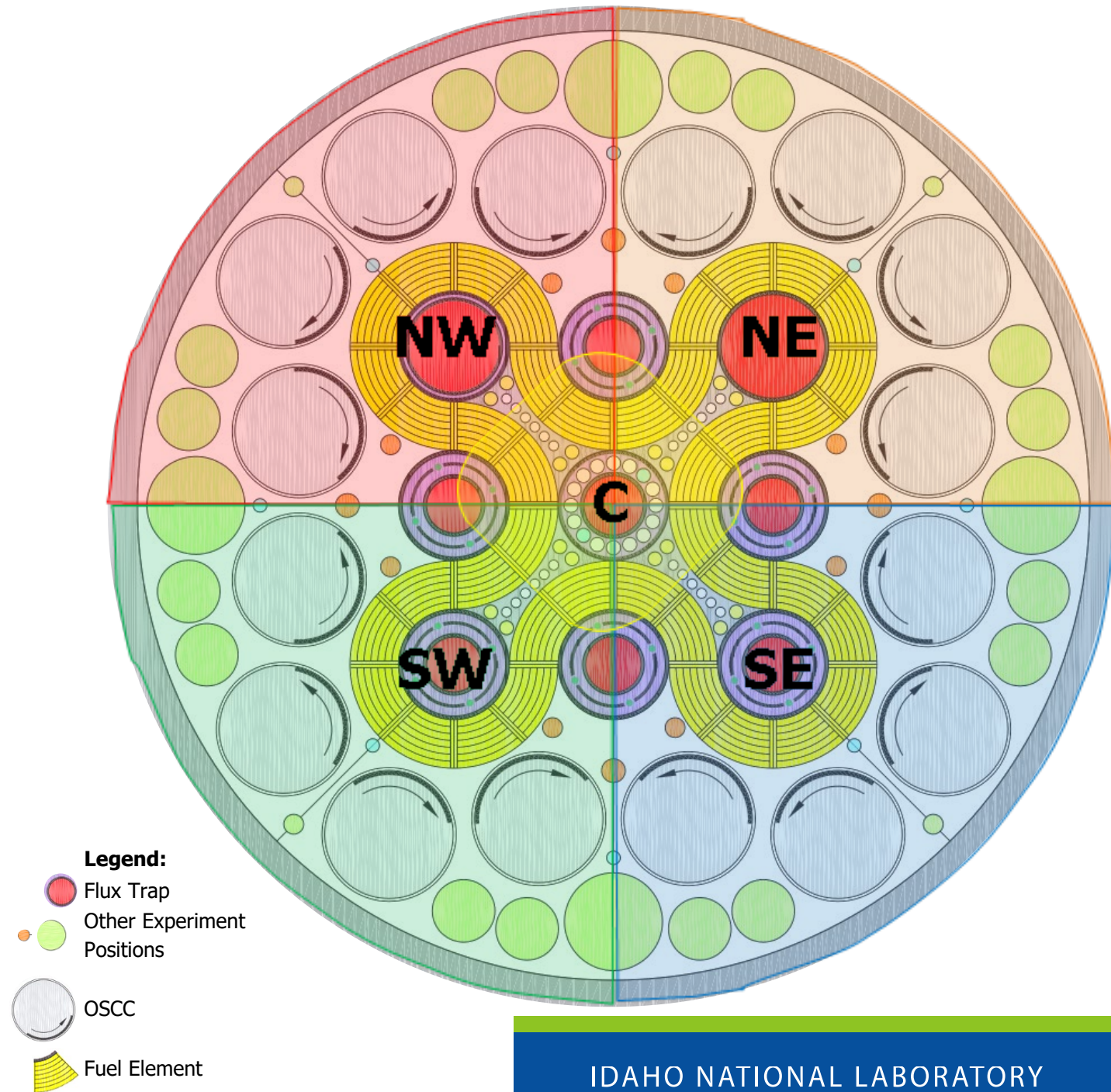
Introduction to ATR

- More than 70 test positions
 - 9 flux traps
 - 6 (of the 9) have loops
 - Independent Chemistry, temperature, and pressure
- Control Elements
 - 6 Safety Rods (annular)
 - 16 Outer Shim Control Cylinders (OSCCs)
 - 22 Neck Shims
 - +2 Regulating Rods
- 40 Fuel Elements
 - 19 plates
 - 48" (120cm) active length
 - Serpentine arrangement



Introduction to ATR

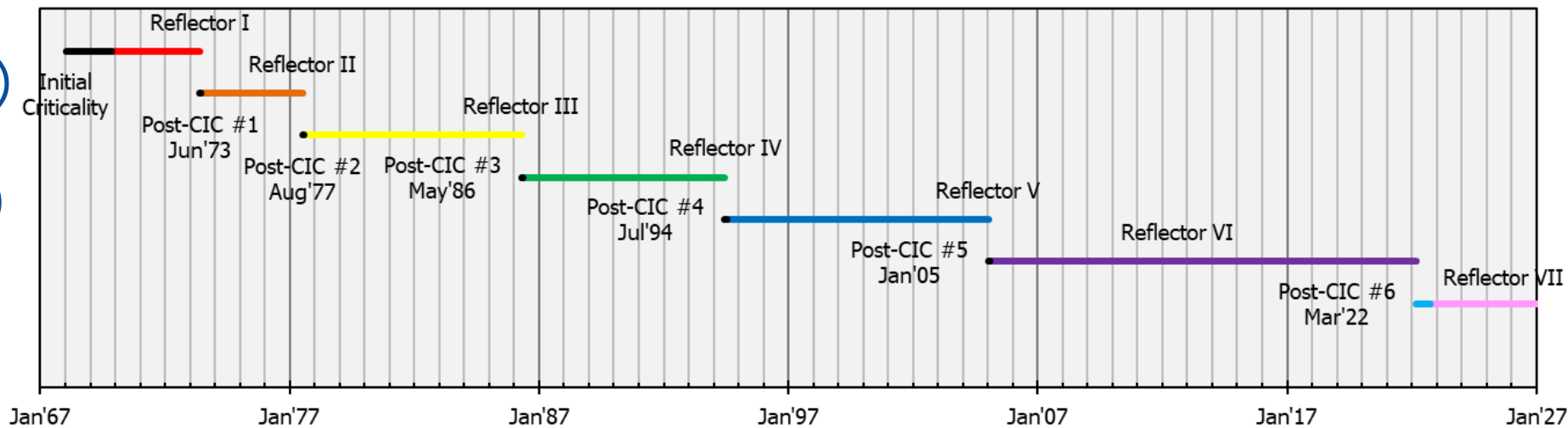
- Design Summary
 - 250 MW_{th} (Typically 110MW_{th})
 - Max thermal neutron flux:
 - 10^{15} n/cm²-s
 - Max fast neutron flux:
 - 5×10^{14} n/cm²-s
- Companion ATRC
 - 5 kW_{th}



Nuclear Testing – Historical

- Performed each CIC:

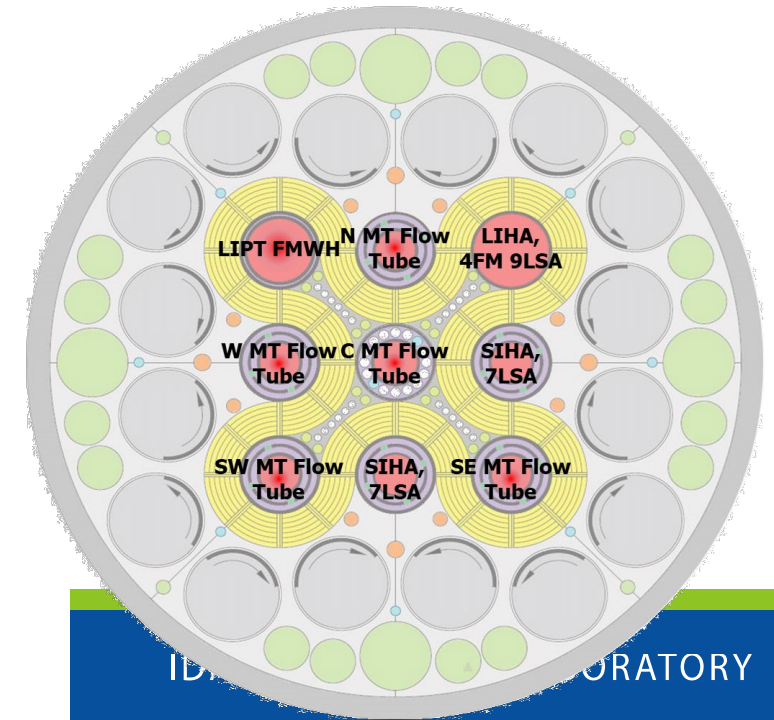
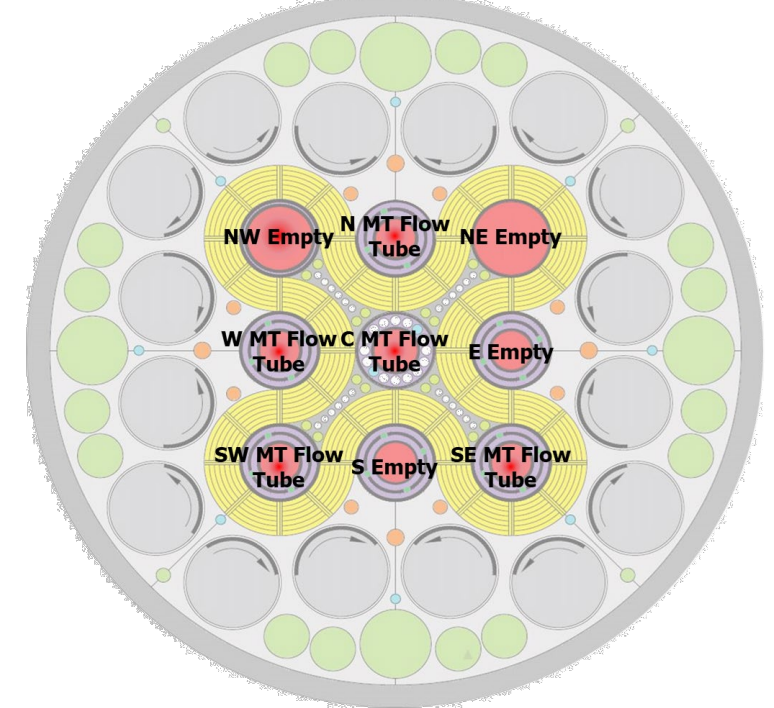
- #1: 1973
(Reflector II)
- #2: 1977
(Reflector III)
- #3: 1986
(Reflector IV)
- #4: 1994
(Reflector V)
- #5: 2004
(Reflector VI)



- Also for initial criticality and core reconfigurations

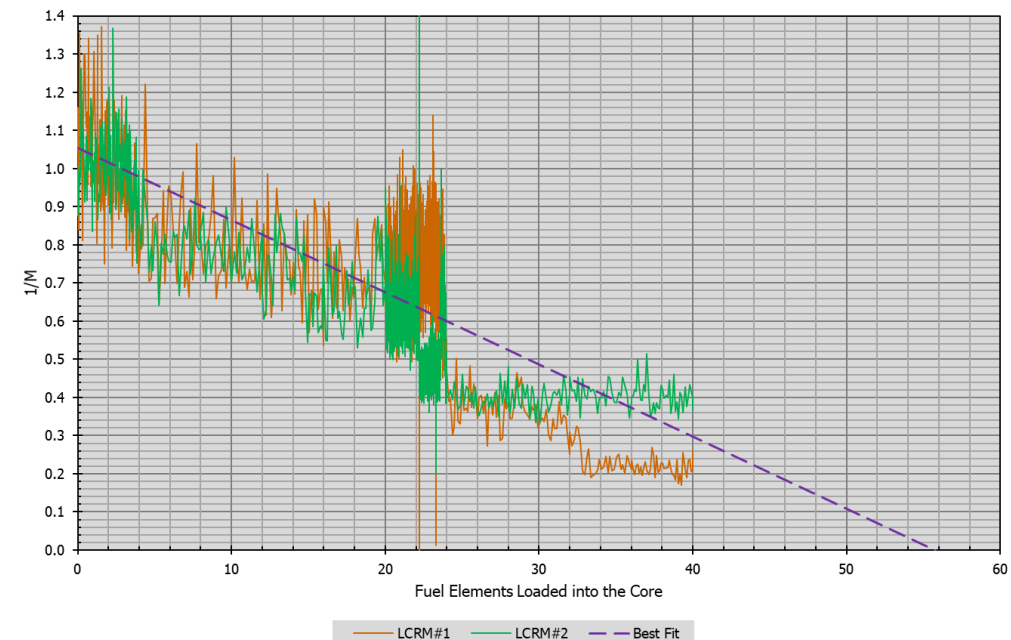
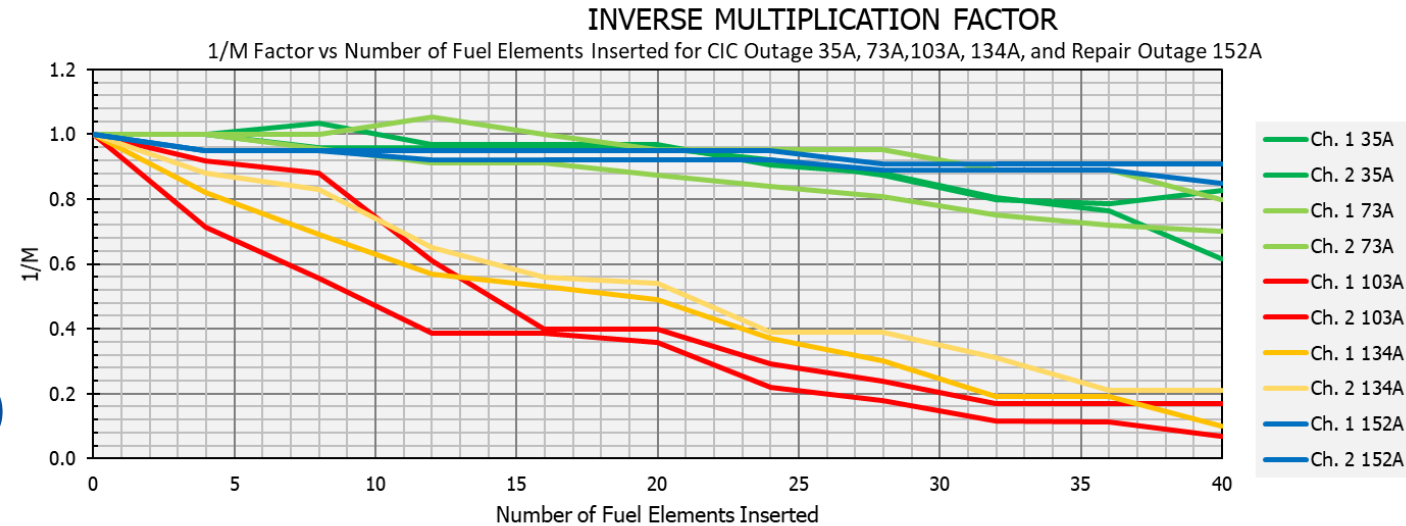
Nuclear Testing – Low-power Tests

Cycle	Tests	Target Power	Experiment Loading
170CIC-1	NT-2	2 ±1 kW	Mostly Water
170CIC-2	NT-3		
170CIC-3	NT-3		2 ± 1 MW
170CIC-4	NT-4, NT-5		
170CIC-5	NT-6		
170CIC-6			
170CIC-7			



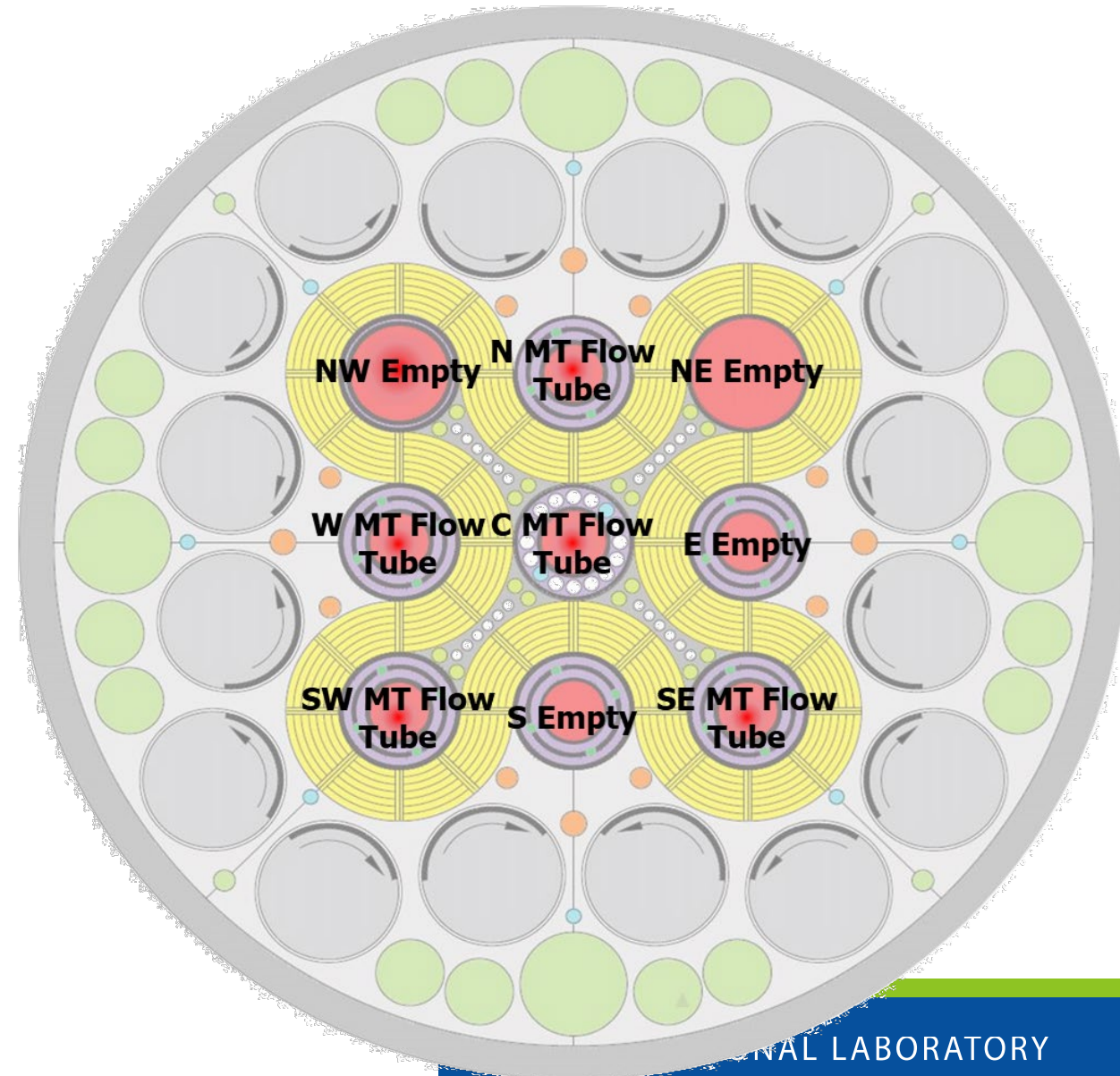
NT-1 – Fuel Loading – Cycle 170CIC-1 Outage

- Validate subcriticality
- Load incrementally
 - 4 FEs in each of 10 steps
 - Startup source (^{124}Sb) for (γ, n) reaction in Be
 - In past, little increase in multiplication w/o source
 - Purple dashed line is a best-fit extrapolation from LCRM data
- Fresh fuel
 - Safer to handle
 - Least uncertainty in modeling



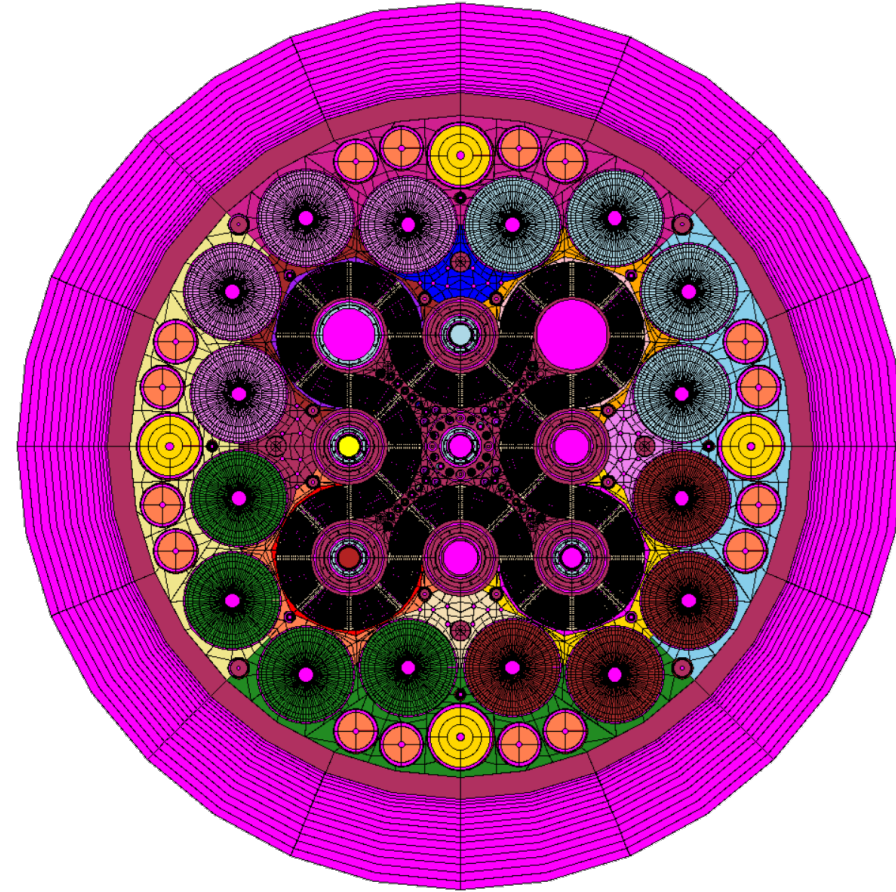
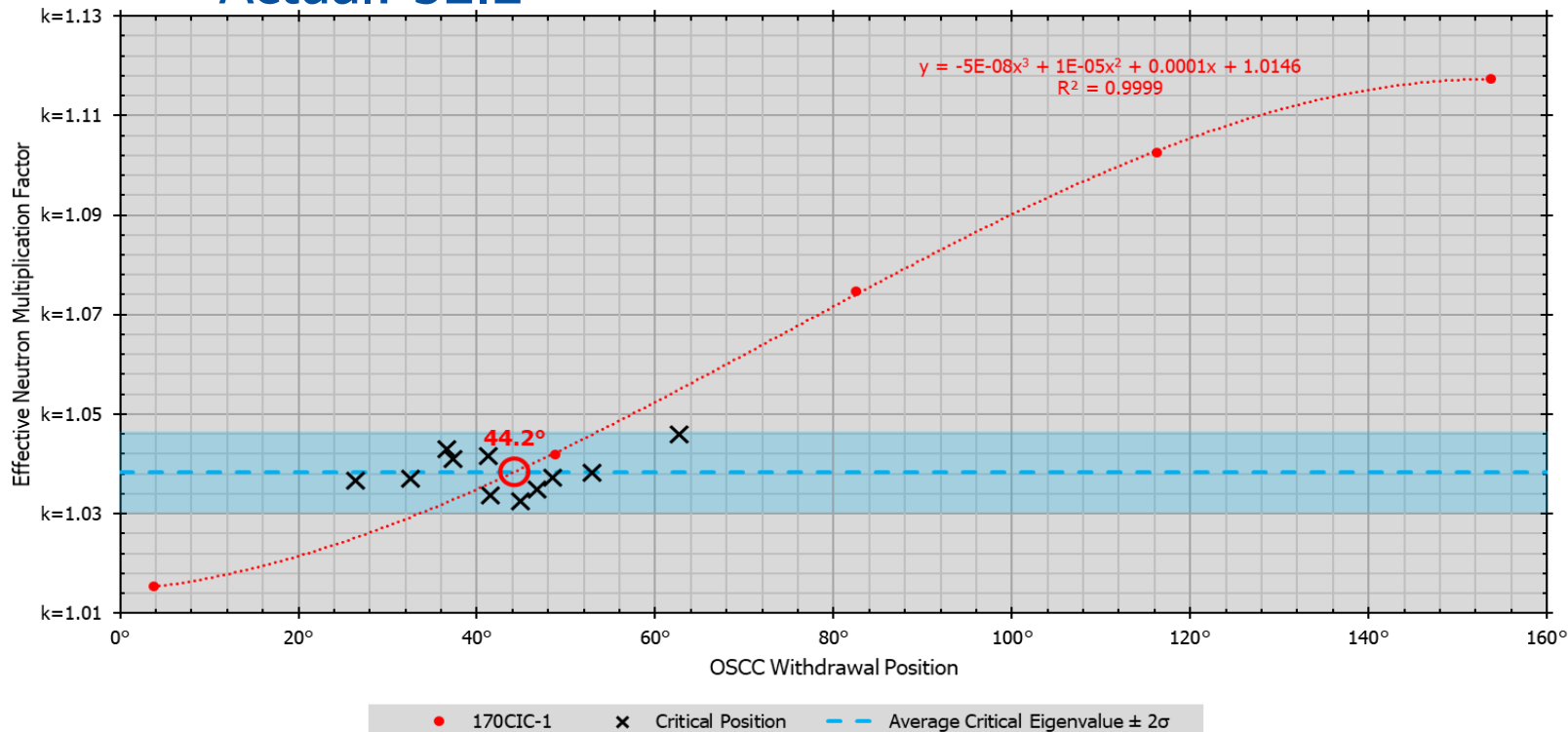
NT-2 – Initial Criticality – Cycle 170CIC-1

- Validate model's representation of core assembly
 - Quantifies holddown reactivity margin
- Calibrate nuclear instruments, to target desired power in NT-3
- Differing experiment loading in the past (all with fresh fuel):
 - 1986: 57.5°
 - 1994: 52.4°
 - 2004: 29.3°
 - 2012: 28.8°
 - 2022: 52.2°



NT-2 – Initial Criticality for Cycle 170CIC-1

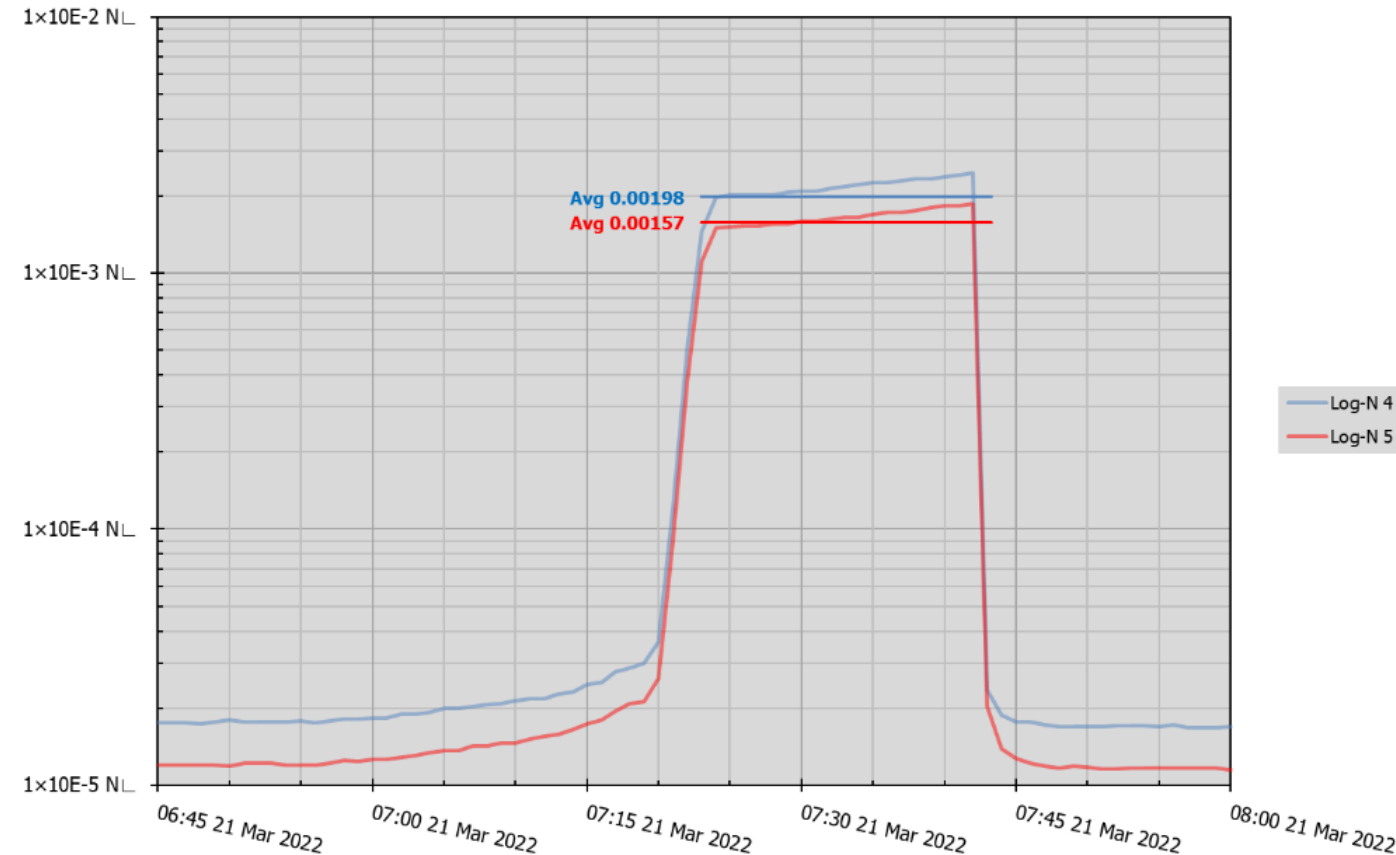
- Model: $\approx 44.6^\circ$
 - Reach Critical Eigenvalue
- ATRC: $\approx 53.2^\circ$
- Actual: 52.2°



NT-2 – Initial Criticality for Cycle 170CIC-1

- Log-N instruments calibrated with Co dosimetry
 - ATRC Measured Power: 0.696kW
 - ATR Power from Co Ratio: 1.6kW
 - Validates Log-N: ≈ 1.8 kW

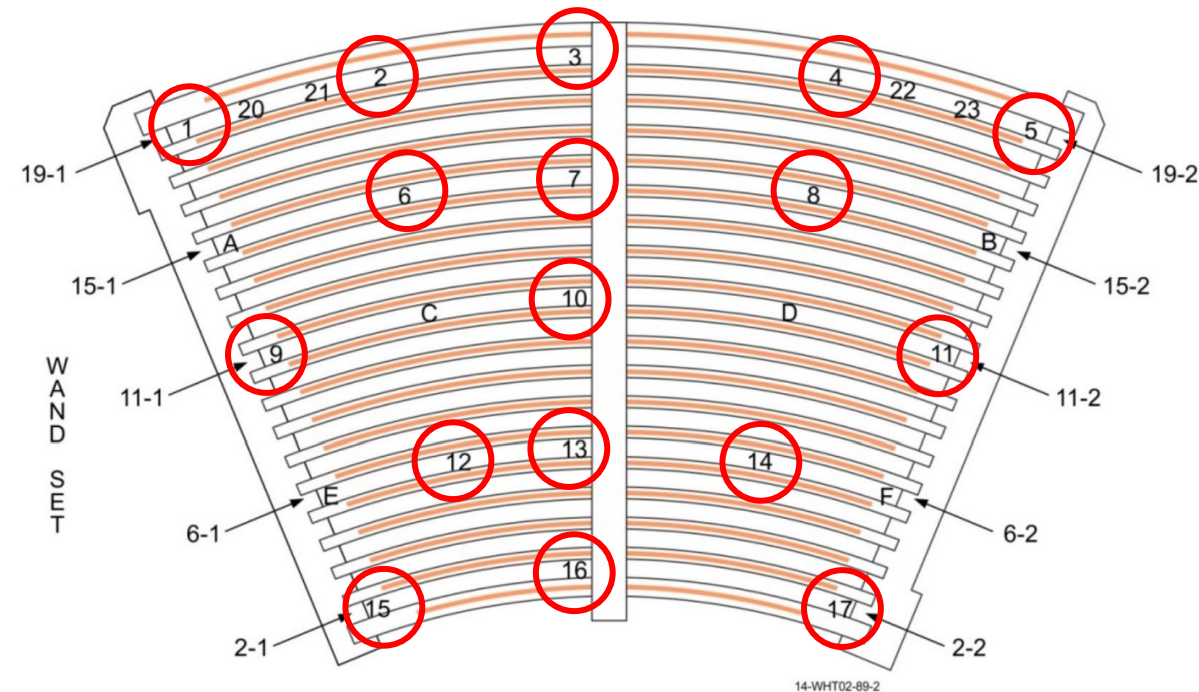
	ATRC	ATR
B-1	0.0670 μ Ci/g	0.169 μ Ci/g
B-2	0.0650 μ Ci/g	0.231 μ Ci/g
B-3	0.1170 μ Ci/g	0.249 μ Ci/g
B-4	0.1020 μ Ci/g	0.273 μ Ci/g
B-5	0.1100 μ Ci/g	0.246 μ Ci/g
B-6		0.142 μ Ci/g
B-7	0.0771 μ Ci/g	0.142 μ Ci/g
B-8	0.0578 μ Ci/g	0.134 μ Ci/g
Average	0.0851 μ Ci/g	0.1983 μ Ci/g
Core Power	0.69602kW	1.6209kW



NT-3 – Power Division Measurement – Cycles 170CIC-2/3

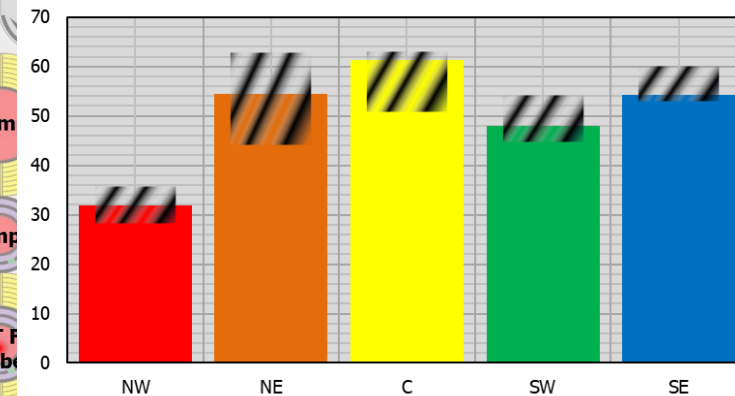
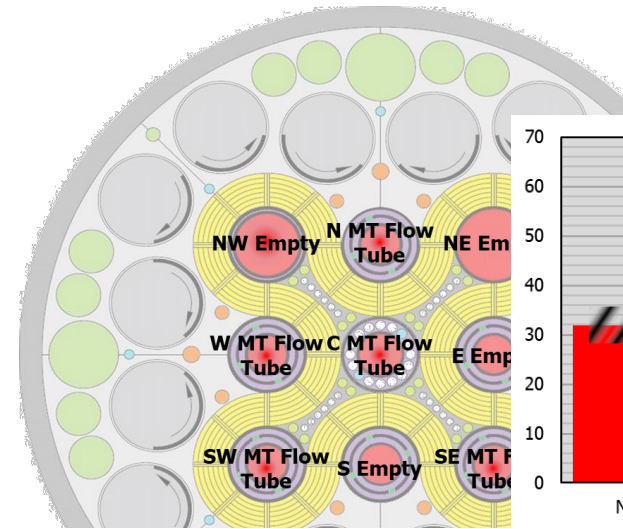
Flux Run Basics

- Load flux wands + fission wires
 - Measure midplane power directly, by activations in U-Al wires
 - 10 wands
 - 17 total wires
 - (20-23 and A-D not used in ATR)
- Exactly 20min irradiation
 - Avoids saturating
 - Start $1/e$ times 2kW
- Frequently performed in ATRC
 - Only real way to know ATRC power

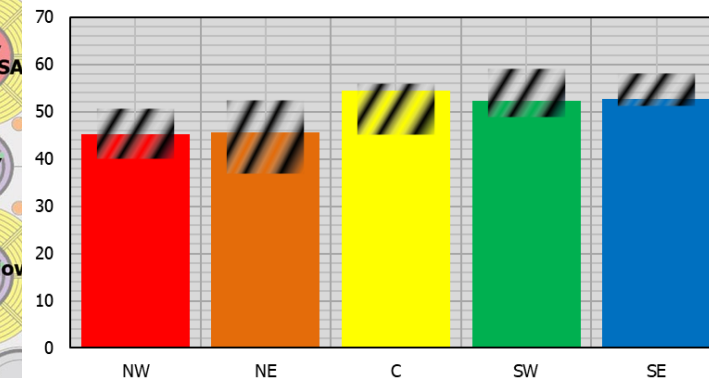
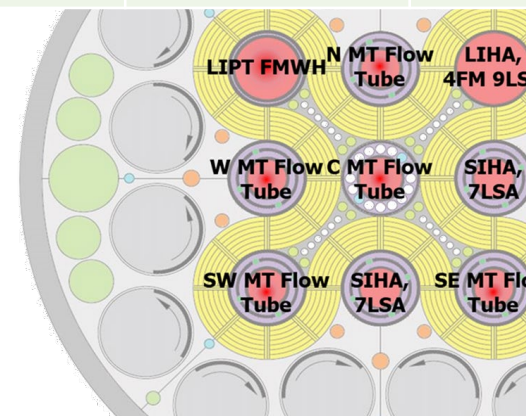


NT-3 – Power Division – Cycles 170CIC-2/3

- Need ____W/Bq/mg for NT-6
 - NT-3 power (kW) is appropriate for fission wires
 - NT-6 power (MW) is needed for installed power indication
 - Calibrate Nb dosimetry in NT-3 for NT-6
 - Previous NT used Ag dosimetry and was often unsuccessful due to competing thermal and fast activations

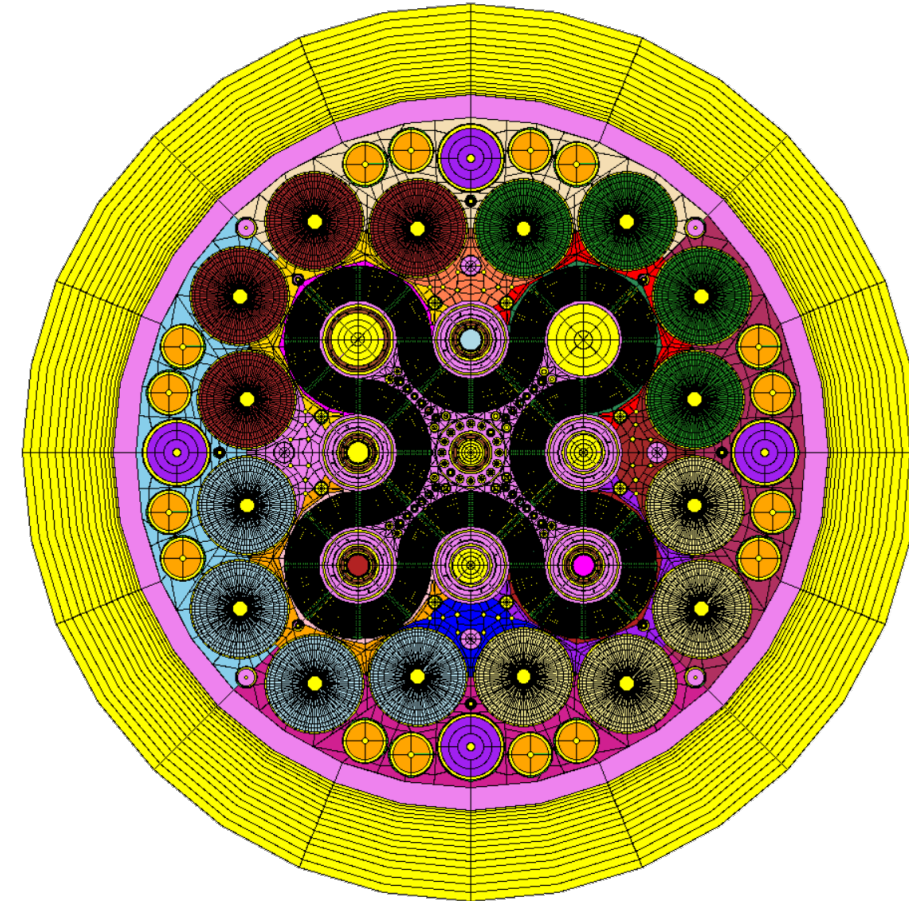
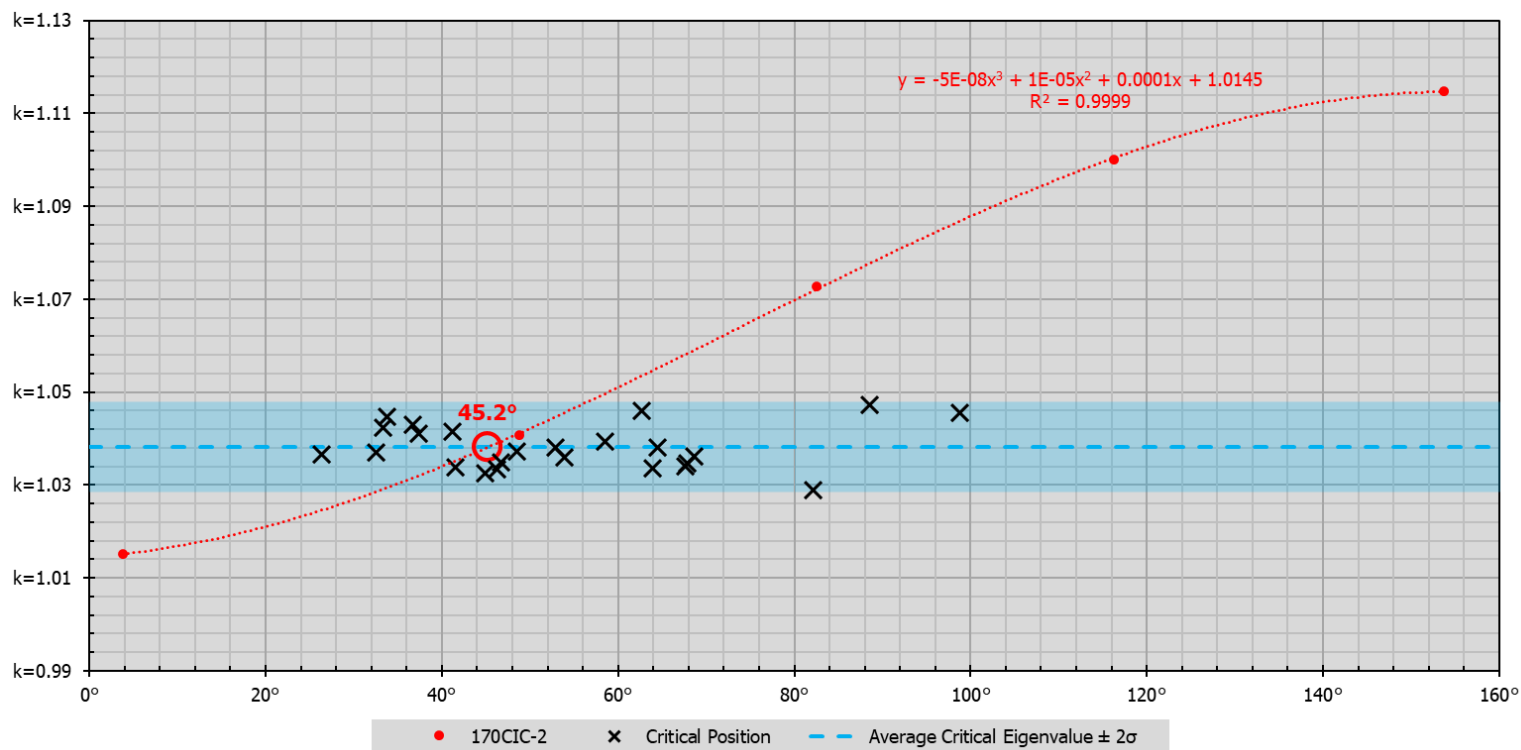


	NW	NE	C	SW	SE
170CIC-2	31.94	54.52	61.28	47.89	54.33
170CIC-3	45.16	45.55	54.47	52.20	52.61



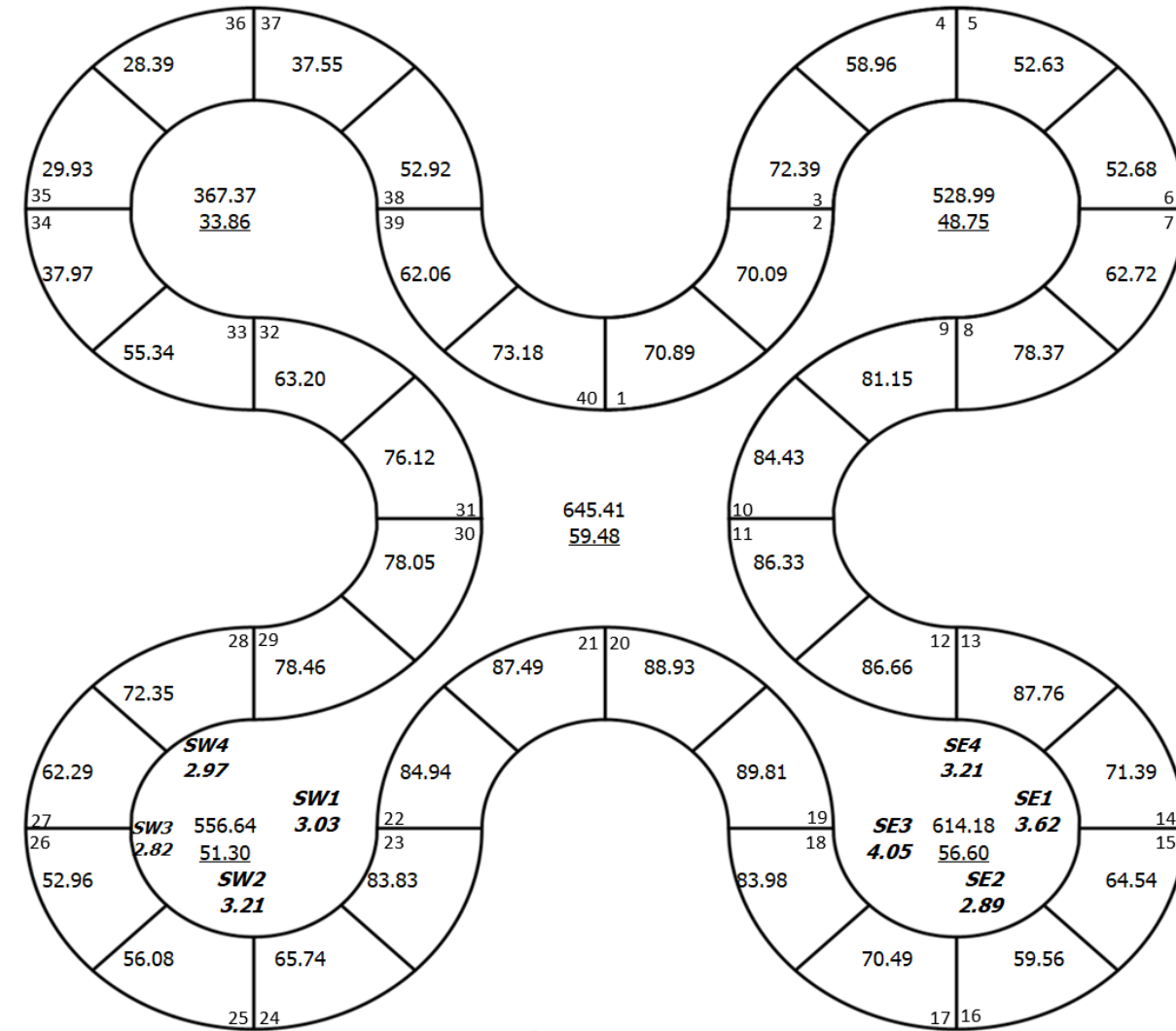
NT-3 – Cycle 170CIC-2

- Model: $\approx 45.2^\circ$
- ATRC: $\approx 57.2^\circ$
- Actual: 57.3°



NT-3 – Power Division Measurement – Cycle 170CIC-2

- 170CIC-2 dosimetry only in SW/SE
 - Dosimetry positions in Safety Rods
- Other lobes' dosimetry requires hardware to be loaded, which interferes with intended water-filled benchmark



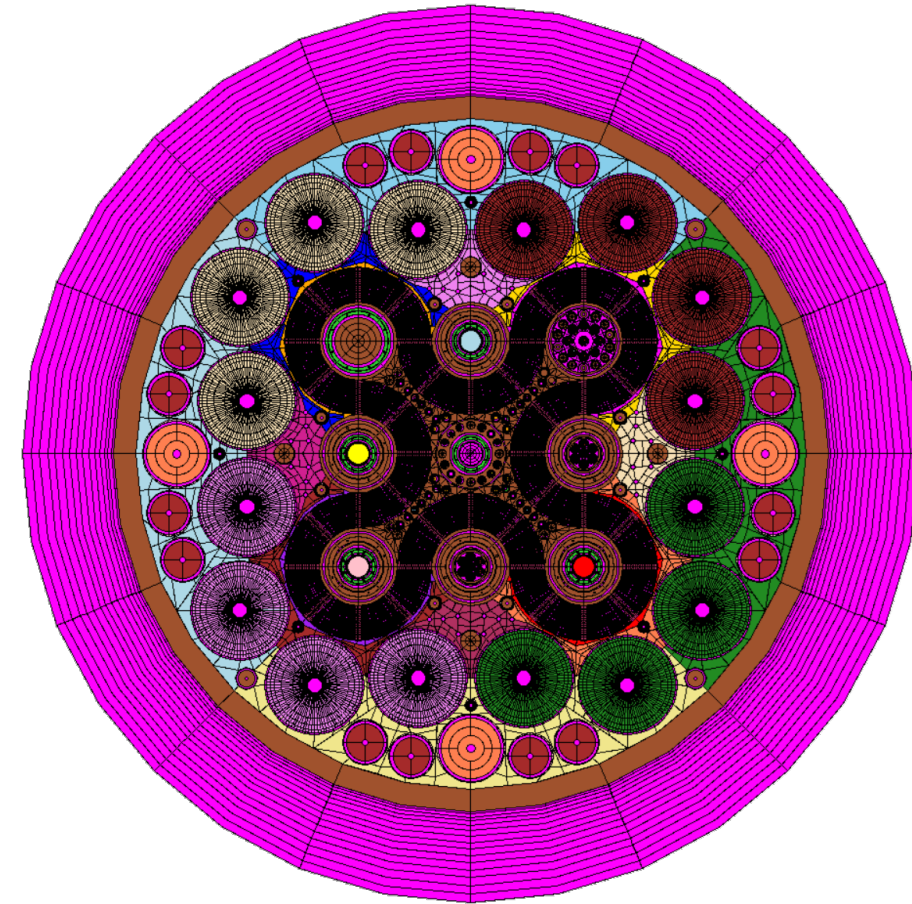
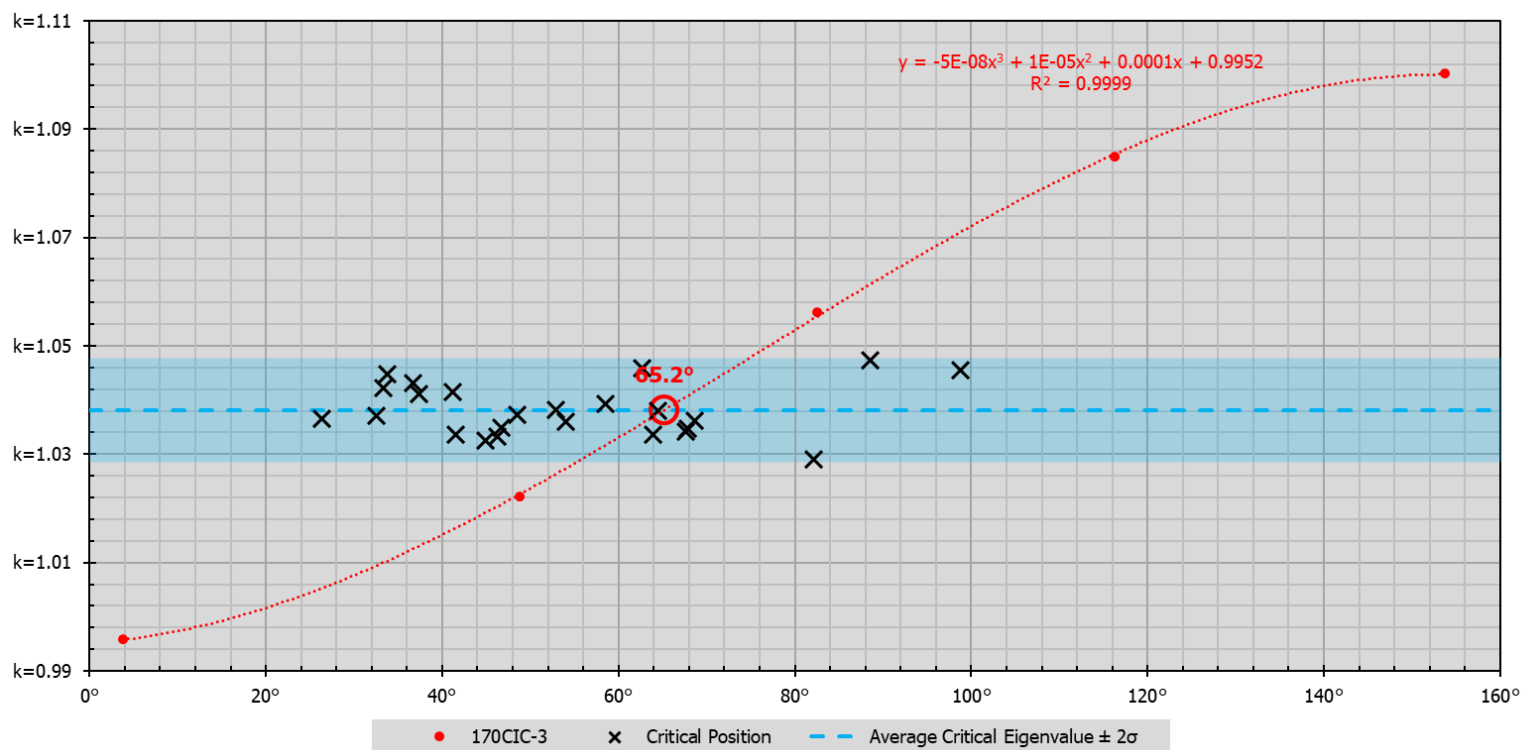
Cycle 170CIC-2

Fuel Element Powers are shown in watts. Lobe Powers are shown within the respective lobes. Underlined lobe powers are normalized to 250 watts.

Total Core Power = 2,712.60 watts

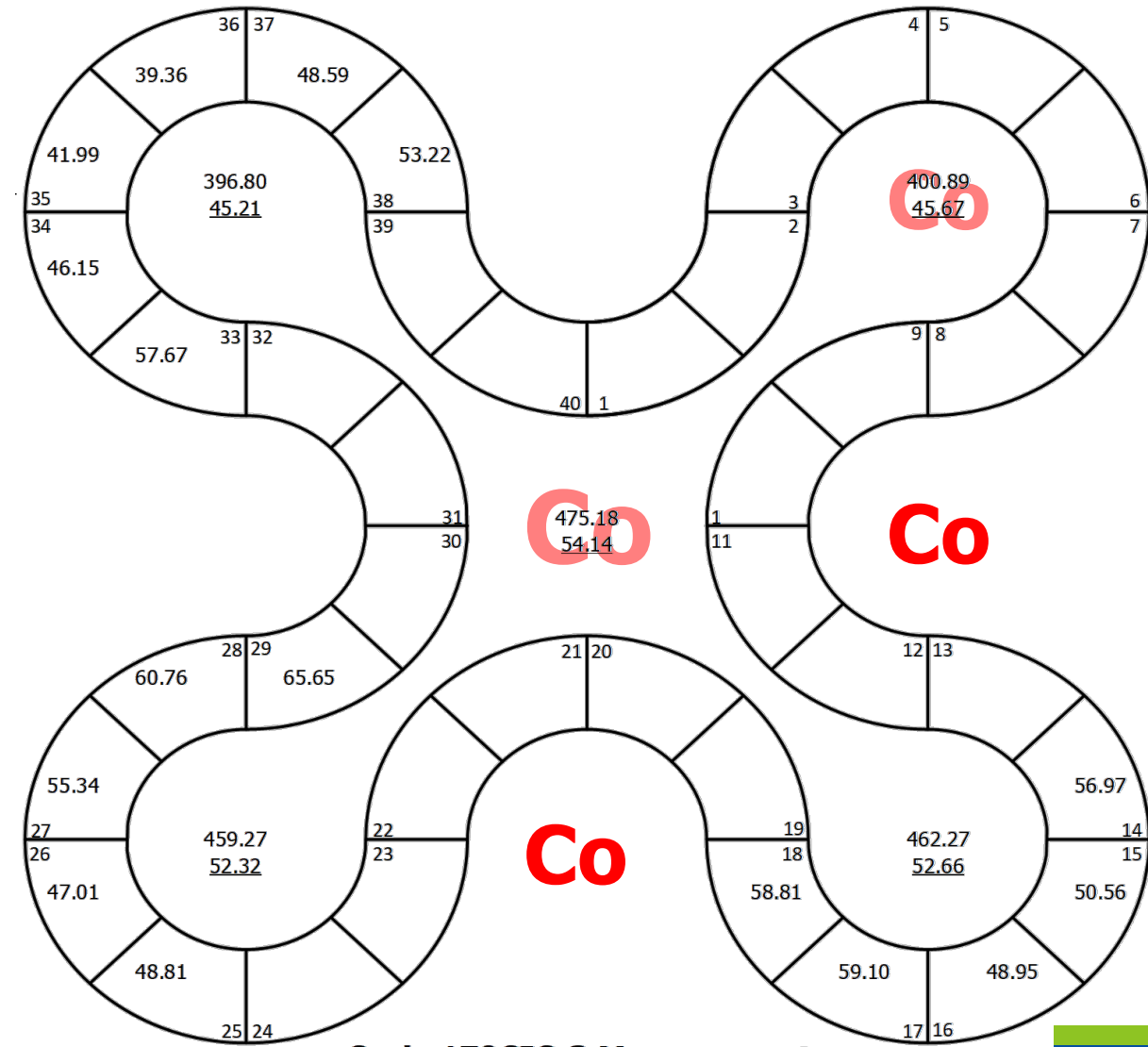
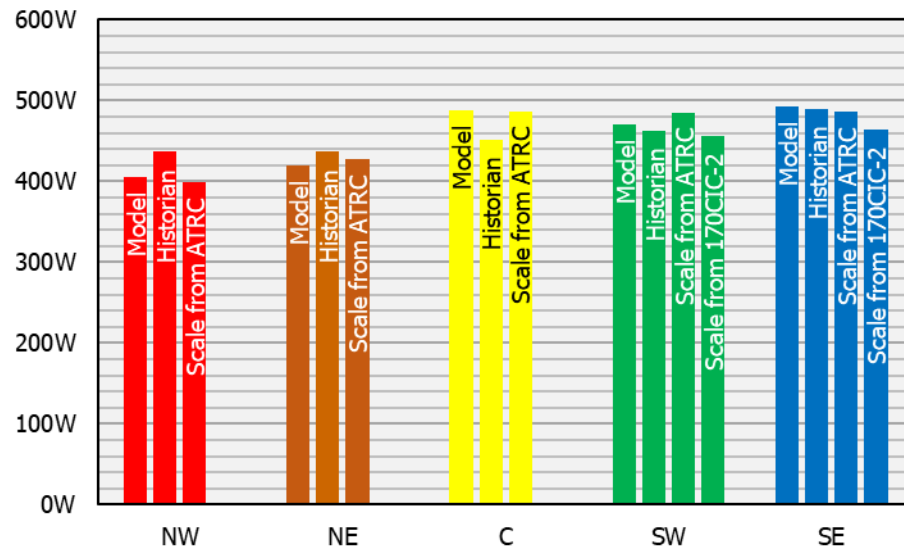
NT-3 – Initial Criticality for Cycles 170CIC-3 through -7

- Model: $\approx 65.2^\circ$
- ATRC: $\approx 63.5^\circ$
- Actual: 65.5°



NT-3 – Power Division Measurement – Cycle 170CIC-3

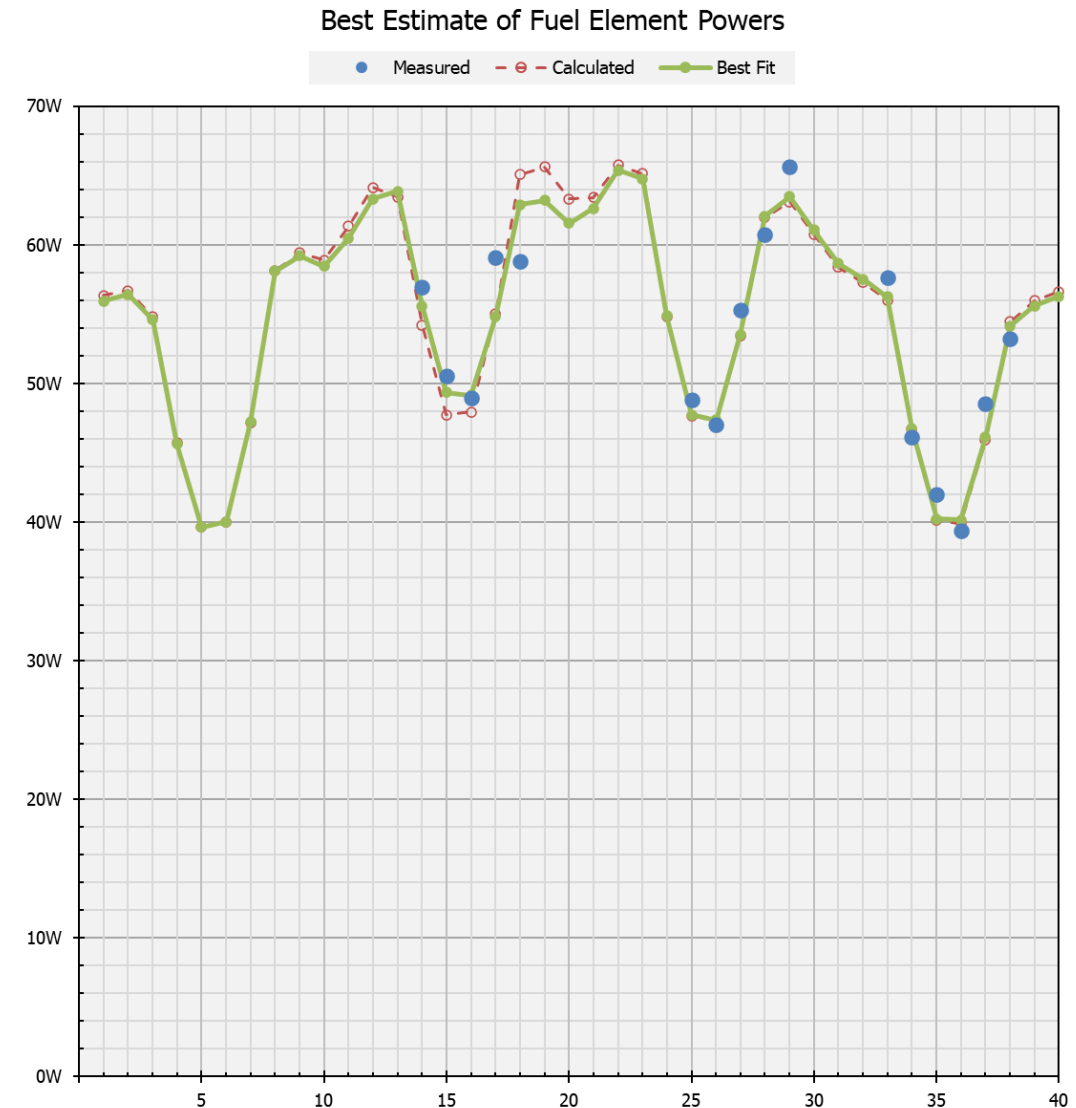
- 170CIC-3 Results:
 - Most fuel element measurements failed
 - Several remaining ways of computing lobe powers



Cycle 170CIC-3 Measurements

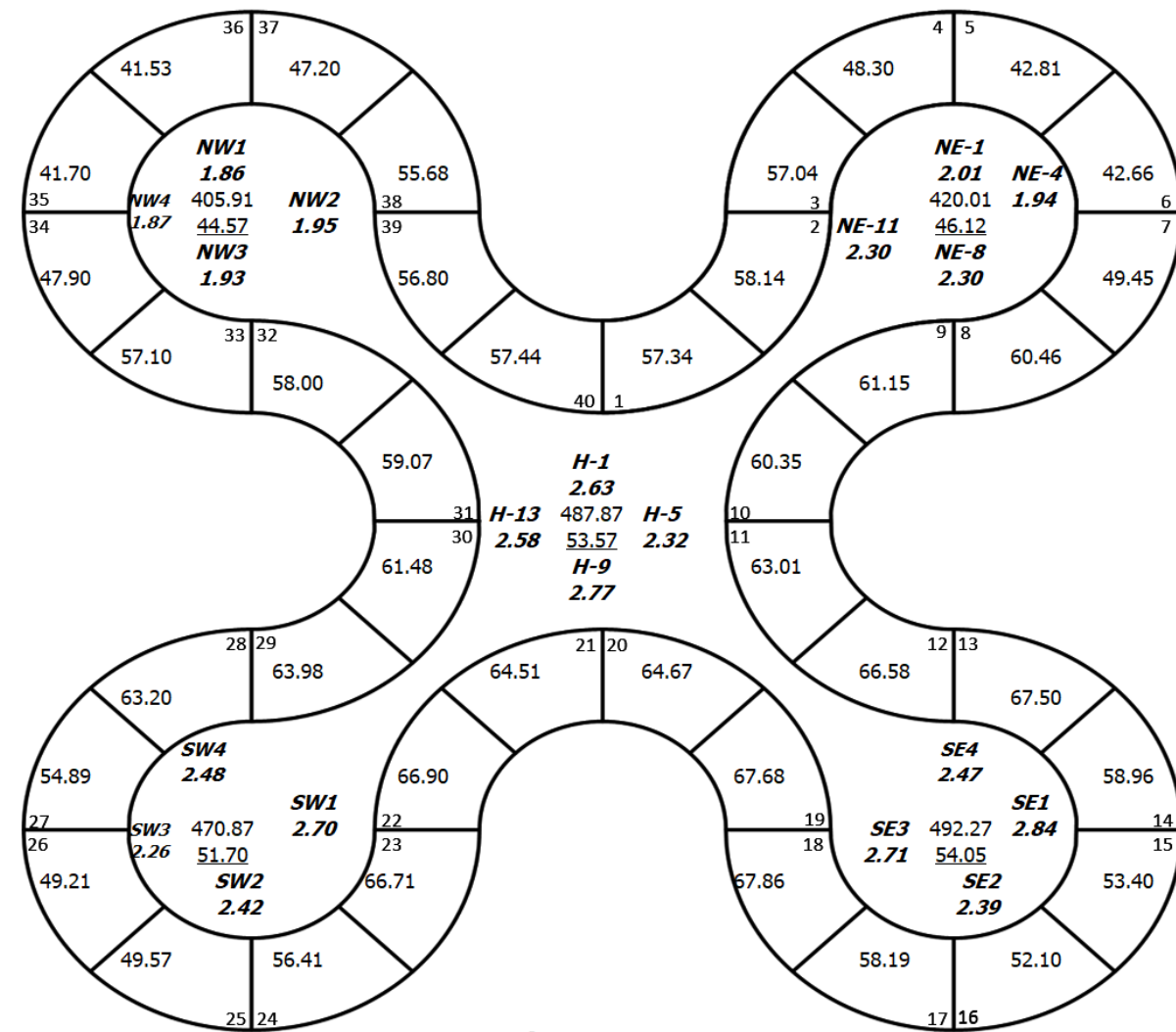
NT-3 – Power Division Measurement – Cycle 170CIC-3

- Modeled FE powers can be adjusted to best-estimate powers
 - Leverages available measurements
 - See J. W. NIELSEN, D. W. NIGG, and A.W. LaPORTA, "A Fission Matrix Based Validation Protocol for Computed Power Distributions in the Advanced Test Reactor," *Nucl. Eng. Des.*, **295**, 615 (2015).
 - This method gives additional credibility to modeled powers



NT-3 – Power Division Measurement – Cycle 170CIC-2

- 170CIC-3 dosimetry in all 5 lobes
- This graphic shows modeled powers, as only a few measurements were successful



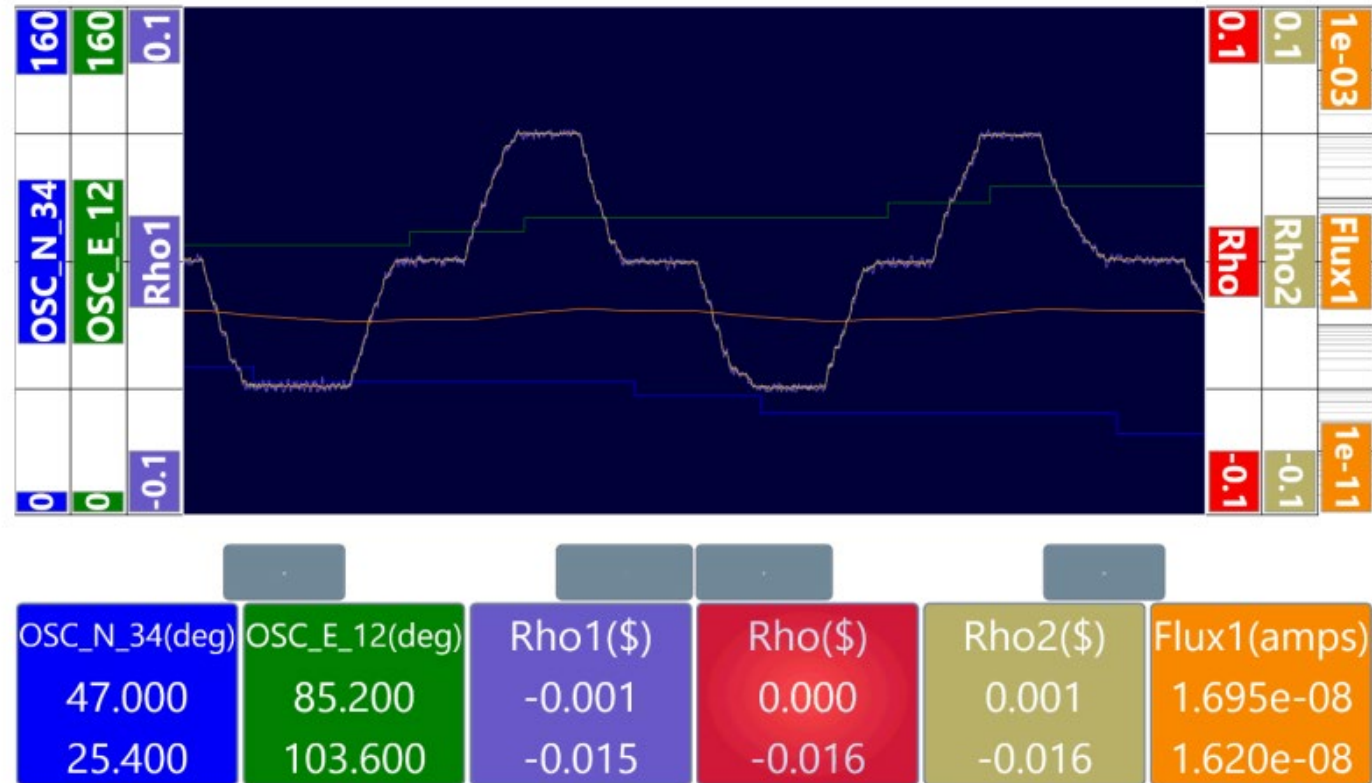
Cycle 170CIC-3

Fuel Element Powers are shown in watts. Lobe Powers are shown within the respective lobes. Underlined lobe powers are normalized to 250 watts.

Total Core Power = 2,276.93 watts

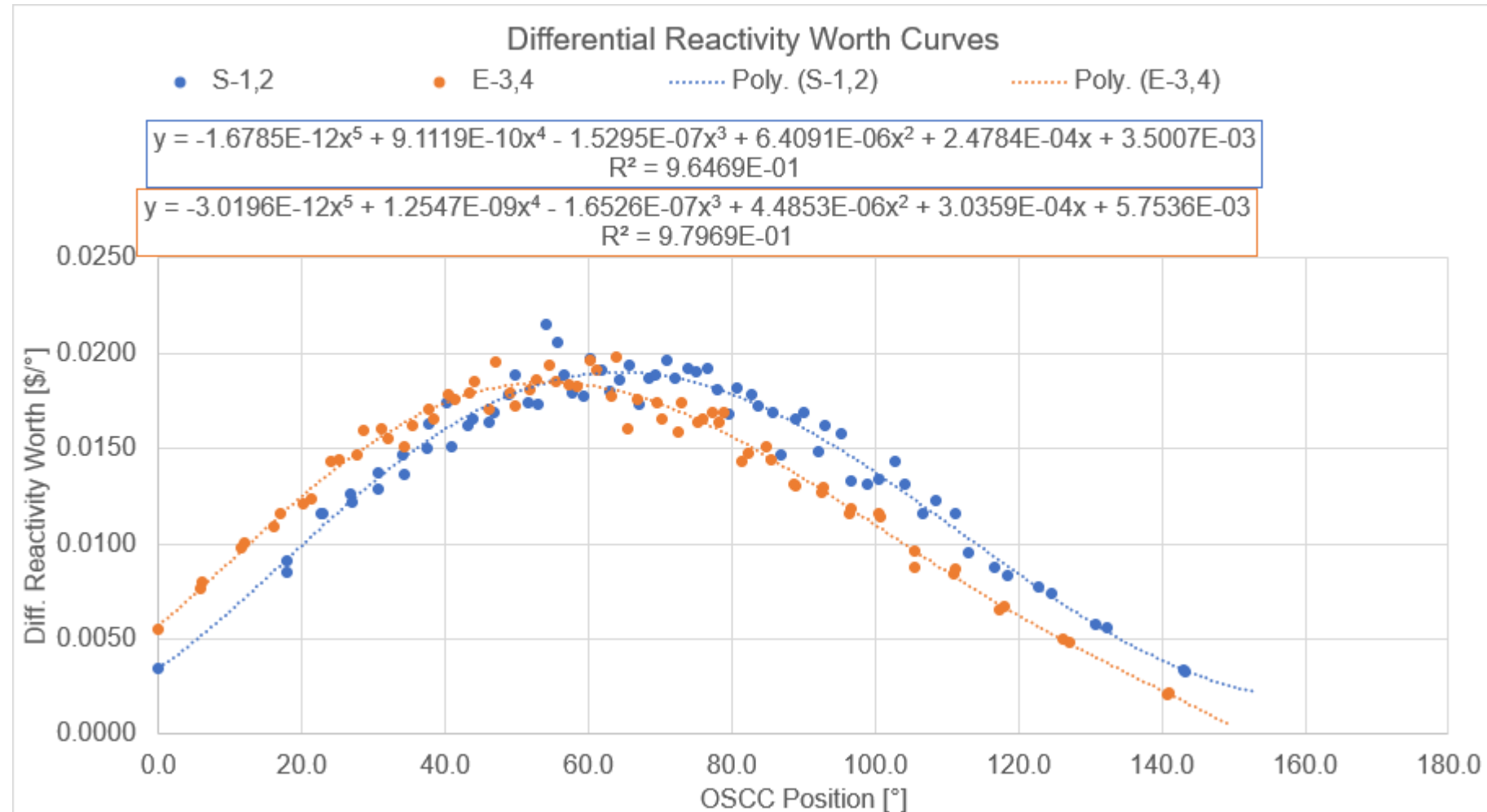
NT-4 – Shim and Coolant Worth Calibrations – Cycle 170CIC-4

- Reactivity Measurement Acquisition System (RMAS) computers
 - Directly indicate core reactivity
- Shim incrementally and track core reactivity
 - Safety Rods
 - OSCCs
 - Neck Shims
 - Regulating Rods
- Primary isothermal temperature coefficient of reactivity
- Loop temperature coefficient of reactivity



NT-4 – Shim and Coolant Worth Calibrations – Cycle 170CIC-4

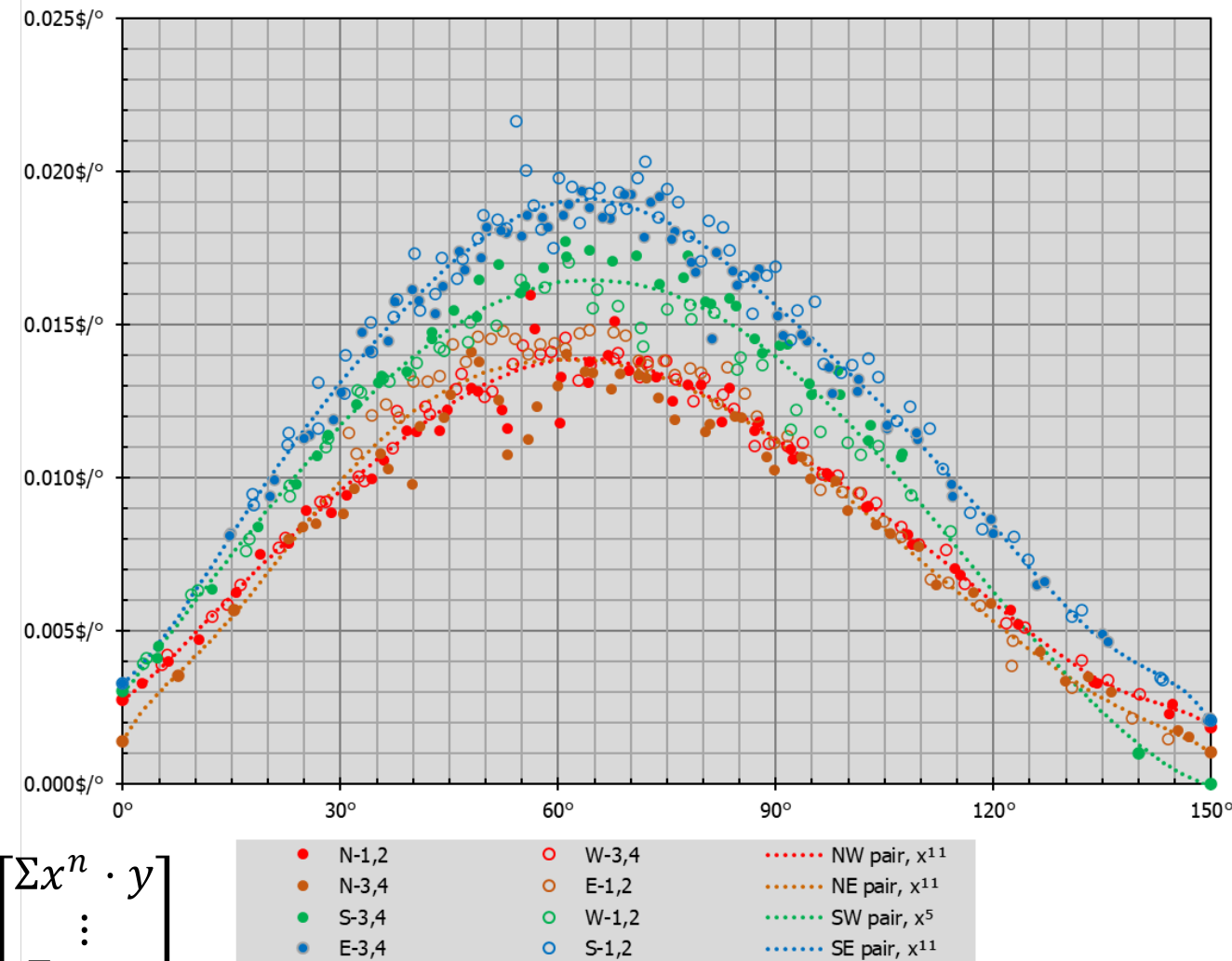
- OSCCs pairs misaligned
 - NE by 6.5°
 - SE by 8.8°
- These curves should lie exactly on top of each other



NT-4 – Shim and Coolant Worth Calibrations – Cycle 170CIC-4

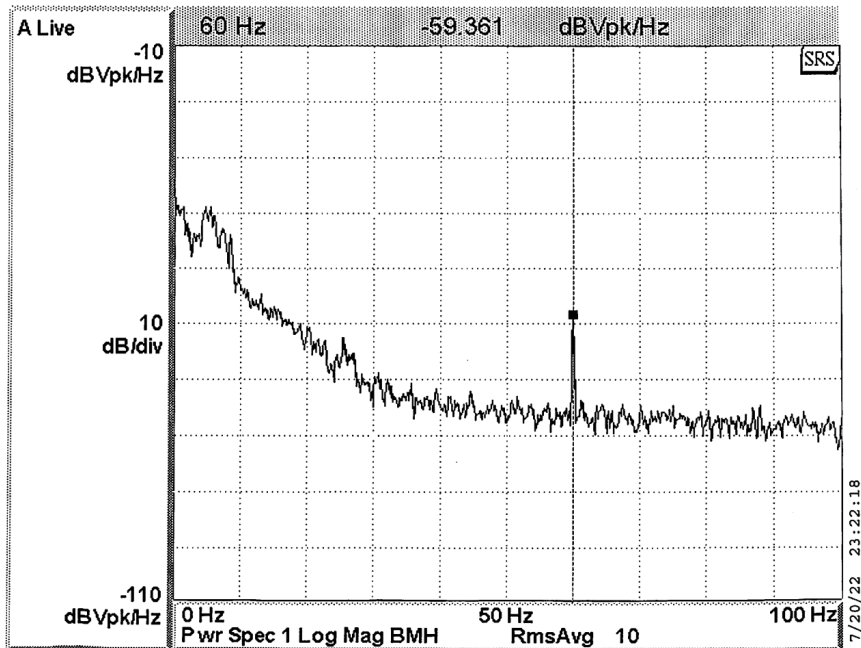
- OSCCs have most data
- Assume the true integral worth curve is strictly monotonic
 - Found highest-order polynomial with
 - 0\$ at 0°
 - >0\$ on (0° - 150°)
 - 1st derivative >0\$/° on (0° - 150°)

$$\begin{bmatrix} \sum x^{2 \cdot n} & \sum x^{2 \cdot n - 1} & \dots & \sum x^{n+1} & \sum x^n \\ \sum x^{2 \cdot n - 1} & \sum x^{2 \cdot n - 2} & \dots & \sum x^n & \sum x^{n-1} \\ \vdots & \vdots & \ddots & \vdots & \vdots \\ \sum x^{n+1} & \sum x^n & \dots & \sum x^2 & \sum x \\ \sum x^n & \sum x^{n-1} & \dots & \sum x & \sum x^0 \end{bmatrix} \times \begin{bmatrix} a_n \\ \vdots \\ a_1 \end{bmatrix} = \begin{bmatrix} \sum x^n \cdot y \\ \vdots \\ \sum x \cdot y \\ \sum y \end{bmatrix}$$

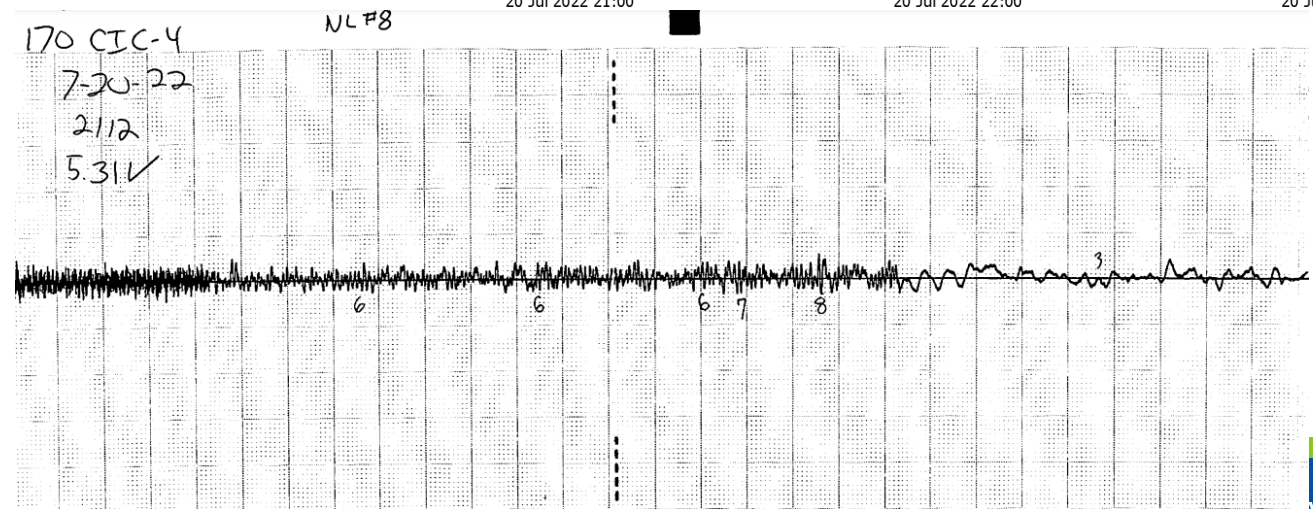
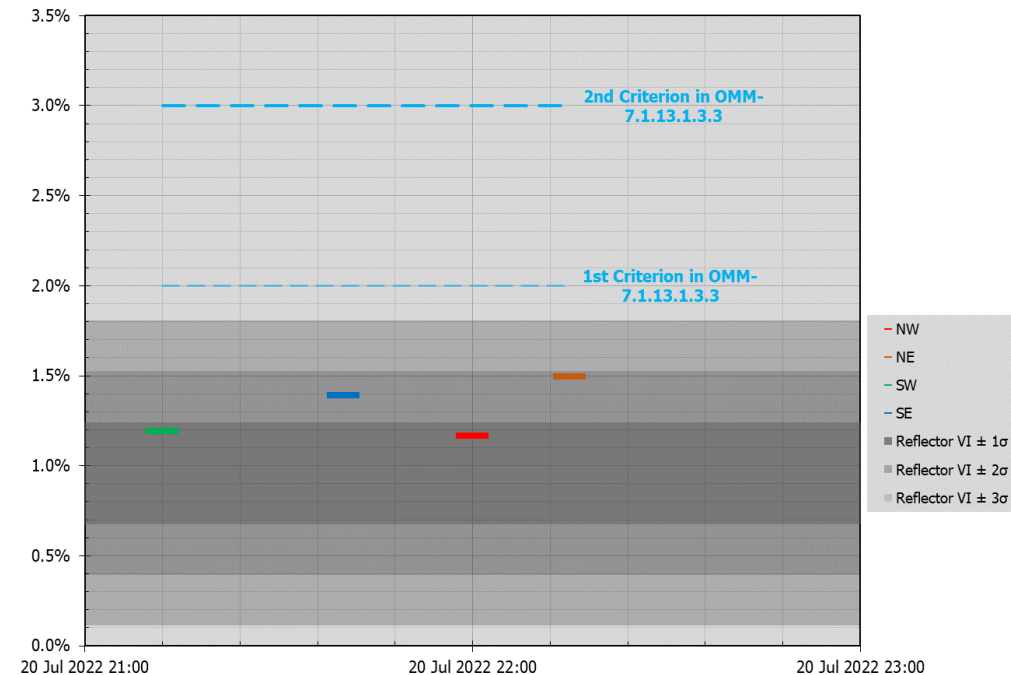


NT-5 – Power Variation – Cycle 170CIC-4

- Power Variation measurement for each quadrant's neutron level instrument
 - All $\pm 2\sigma$ from expectation
- Power spectral density for SW
 - Expected peaks for known hardware



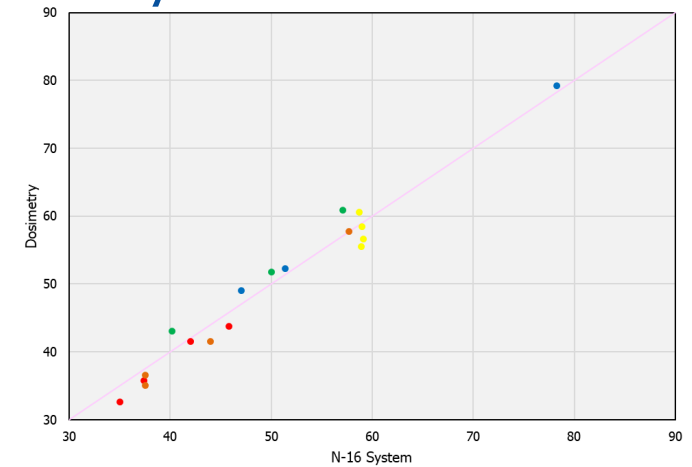
Power Variation



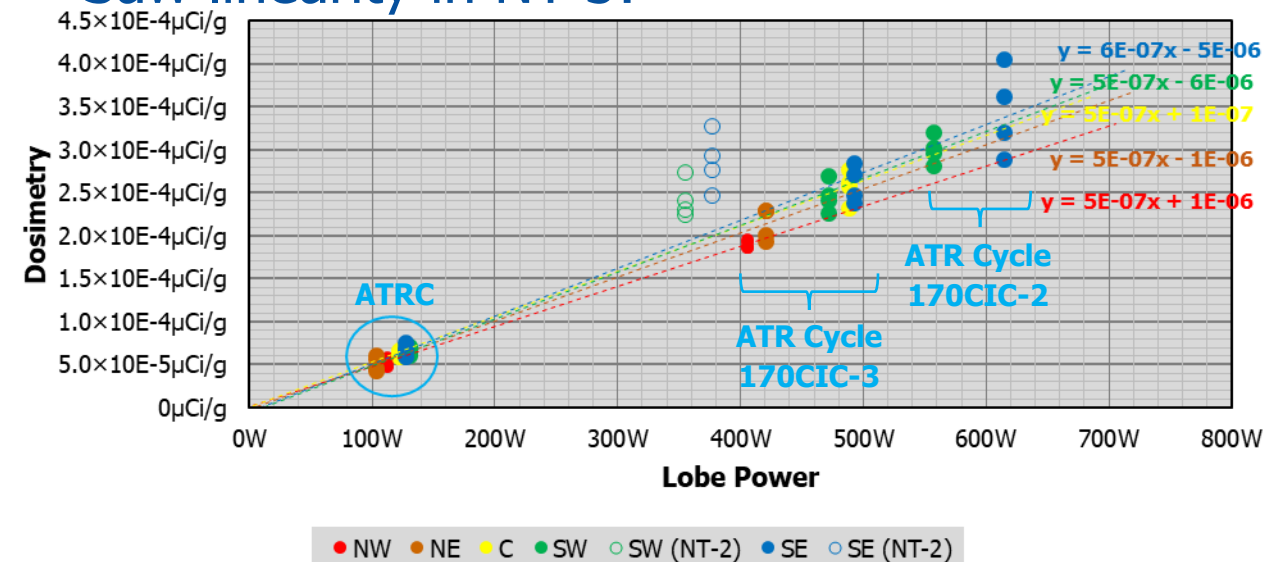
NT-6 – N-16 Calibration – Cycles 170CIC-5+

- Need ____W/Bq/mg from 170CIC-3
- Flux Trap dosimetry needed in NT-6
- Whereas 170CIC-3 is balanced OSCCs
 - Cycle 170CIC-5: balanced
 - Failed due to incorrect dosimetry
 - Cycle 170CIC-6: push toward S
 - Cycle 170CIC-7: repeat failed 170CIC-5

Hope to see linearity in NT-6:



Saw linearity in NT-3:



Power Escalation Tests NT-7 – NT-12 – Cycle 171A-1

- Cycle 171A-1 is a normal cycle
 - Sponsored experiments
 - Designed Fuel loading
- 3 tests are normal parts of reactor startup
 - NT-7: critical shim prediction
 - NT-8: comparison of power division to prediction
 - NT-10: power variation data
- 1 test takes dedicated time
 - NT-9 OSCC calibrations
 - Validate misalignment corrections from NT-4
- 2 test are done in parallel with experiment irradiation
 - NT-11 At-power loop temperature coefficient of reactivity measurement
 - NT-12 N-16 multiplier characterizations

Cycle	Power Escalation Test	Corresponding Low-power Test
171A-1	NT-7	NT-2
	NT-8	NT-3
	NT-9	NT-4
	NT-10	NT-5
	NT-11	None
	NT-12	