

## HTGR Validation: NEUP Survey and Database Database Development for HTGR Thermal-Fluid Experiments

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Department of Advanced Reactor Technology & Design Idaho National Laboratory

## **HTGR Validation: NEUP Survey and Database**

Database Development for HTGR Thermal-Fluid Experiments

DOE ART Gas-Cooled Reactor (GCR) Review Meeting Virtual Meeting
July 25 – 27, 2023



#### Introduction





- From FY2009 to FY2023, there are in total 35 DOE NEUP [1] projects focusing on the thermal-fluid experiments related with High-Temperature Gas-cooled Reactor (HTGR), producing a large amount of high-quality validation data, however,
  - data is distributed at universities and has not been disseminated to the HTGR community well,
  - final reports are now only available on the OSTI webpage.
  - This is a missed opportunity for the HTGR research community needing code validation data.
- Our work is aimed to improve access to the HTGR validation data and optimize the return on the significant investment made by DOE. Supported by the Advanced Reactor Technologies (ART) Gas-Cooled Reactor (GCR) program [2], we conducted a survey to:
  - Assess completed and ongoing NEUP-funded HTGR-TH projects,
  - With the aim to develop a public-accessible data platform that can be used to retrieve code validation data and guide future NEUP investments.

# **DOE NEUP Project List for HTGR-TH (FY09 – 23)**

Project No.	Project Name	Project Instructors and Affiliation
09-771	Creation of a Full-core HTR Benchmark with the Fort St. Vrain Initial Core and Assessment of Uncertainties in the FSV Fuel Composition and Geometry	William Martin (University of Michigan)
09-781	Experimental Studies of NGNP Reactor Cavity Cooling System With Water	Michael Corradini (University of Wisconsin)
09-784	Investigation of Countercurrent Helium-air Flows in Air-ingress Accidents for VHTRs	Xiaodong Sun (The Ohio State University, now at UMich)
09-817	CFD Model Development and Validation for High Temperature Gas Cooled Reactor Cavity Cooling System (RCCS) Applications	Yassin Hassan (Texas A&M University)
09-830	Graphite Oxidation Simulation in HTR Accident Conditions	Mohamed El-Genk (University of New Mexico)
09-840	Investigation on the Core Bypass Flow in a Very High Temperature Reactor	Yassin Hassan (Texas A&M University)
11-3079	Thermal-hydraulic analysis of an experimental reactor cavity cooling system with air	Michael Corradini (University of Wisconsin)
11-3081	Transient mixed convection validation for NGNP	Barton Smith (Utah State University)
11-3218	Experimental Investigation of Convection and Heat Transfer in the Reactor Core for a VHTR	Masahiro Kawaji (City College of New York)
12-3582	Experimentally Validated Numerical Models of Non-isothermal Turbulent mixing in High Temperature Reactors	Mark Kimber (University of Pittsburgh, now at TAMU)
12-3759	Experimental and CFD Studies of Coolant Flow Mixing within Scaled Models of the Upper and Lower Plenum of a NGNP Gas-Cooled Reactors	Yassin Hassan (Texas A&M University)
13-4884	Validation data for depressurized and pressurized conduction cooldown, validation data acquisition in HTTF during PCC events	Philippe Bardet (The George Washington University)
13-4953	Experimental and Computational Investigations of Plenum-to-Plenum Heat Transfer and Gas Dynamics under Natural Circulation in a Prismatic Very High Temperature Reactor	Muthanna Al-Dahhan (Missouri University of Science & Technology)
13-5000	Model Validation using novel CFD-grade experimental database for NGNP Reactor Cavity cooling systems with water and air	Annalisa Manera (University of Michigan)
14-6435	Fluid stratification separate effects analysis, testing and benchmarking	Andrew Klein (Oregon State University)
14-6786	Experimental Investigation and CFD Analysis of Steam Ingress Accidents in HTGRs	Xiaodong Sun -> Richard Christensen (The Ohio State University)
14-6794	Scaling Studies for Advanced High Temperature Reactor Concepts	Brian Woods (Oregon State University)
15-8205	Experimental investigation of forced convection and natural circulation cooling of a VHTR core under normal operation and accident scenarios	Masahiro Kawaji (City College of New York)
15-8627	Experimental validation data and computational models for turbulent mixing of bypass and coolant jet flows in gas-cooled reactors	Mark Kimber (University of Pittsburgh, now at TAMU)

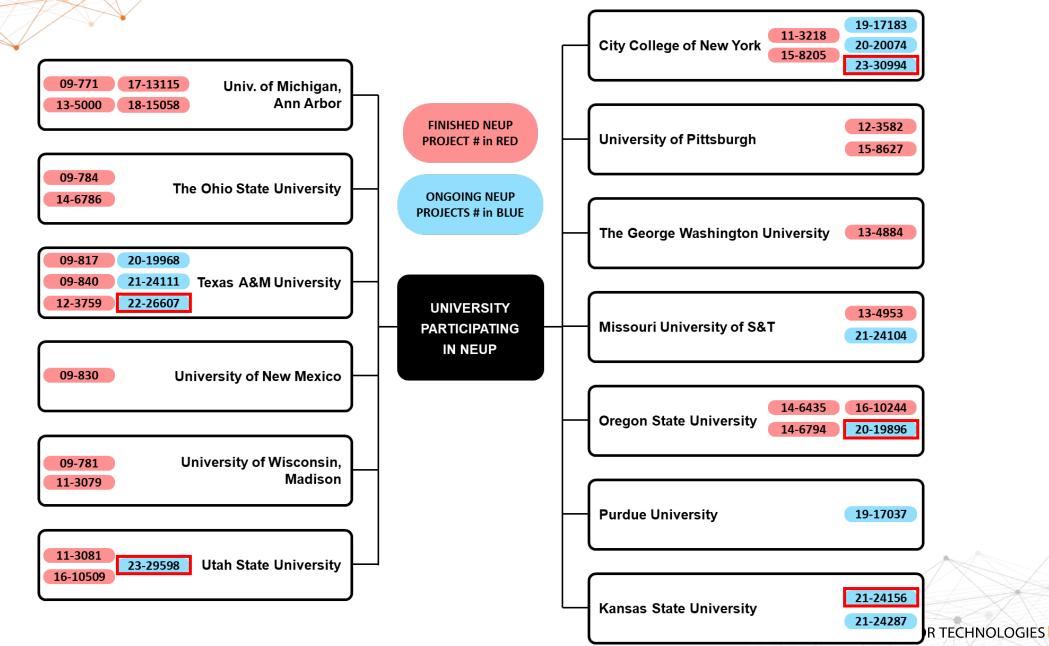
# DOE NEUP Project List for HTGR-TH (cont'd)

Project No.	Project Name	Project Instructors and Affiliation
16-10244	Integral System Testing for Prismatic Block Core Design HTGR	Brian Woods (Oregon State University)
16-10509	CFD and system code benchmark data for plenum-to-plenum flow under natural, mixed and forced circulation conditions	Barton Smith (Utah State University)
17-13115	Experimental Determination of Helium Air Mixing in Helium Cooled Reactor	Victor Petrov (University of Michigan)
18-15058	High-resolution experiments for extended LOFC and Steam Ingress Accidents in HTGRs	Xiaodong Sun (University of Michigan)
19-17037	Investigation of HTGR Reactor Building Response to a Break in Primary Coolant Boundary	Shripad Revankar (Purdue University)
19-17183	Mixing of helium with air in reactor cavities following a pipe break in HTGRs	Masahiro Kawaji (City College of New York)
20-19896	Progression of High Resolution SET and IET Benchmarks on PCC and DCC events in HTGRs	Izabela Gutowska (Oregon State University)
20-19968	Experimental Investigations and Numerical Modeling of Near-wall and Core Bypass Flows in Pebble Bed Reactors	Thien Nguyen, Victor Ugaz, Yassin Hassan (Texas A&M University)
20-20074	Characterization of Plenum to Plenum Natural circulation flows in a high temperature gas reactor (HTGR)	Masahiro Kawaji (City College of New York)
21-24104	Thermal Hydraulics Investigation of Horizontally Oriented Layout micro HTGRs Under Normal Operation and PCC Conditions Using Integrated	Muthanna Al-Dahhan (Missouri University of Science and Technology)
21-24111	Experimental Investigations of HTGR Fission Product Transport in Separate-effect Test Facilities Under Prototypical Conditions for Depressurization and Water-ingress Accidents	N.K. Anand (Texas A&M University)
21-24156	Experimental thermofluidic validation of TCR fuel elements using distributed temperature and flow sensing	Hitesh Bindra (Kansas State University → Purdue University)
21-24287	Investigating Heat Transfer in Horizontally Oriented HTGR under normal and PCC conditions	Hitesh Bindra (Kansas State University → Purdue University)
22-26607	An Innovative Monitoring Technology for the Reactor Vessel of Micro-HTGR	Lesley Wright (Texas A&M Engineering Experiment Station)
23-29598	Uncertainty Quantification of Model Extrapolation in Neural Network-informed Turbulent Closures for Plenum Mixing in HTGRs	Som Dutta (Utah State University)
23-30994 (IRP)	Exascale Simulation of Thermal-Hydraulics Phenomena in Advanced Reactors and Validation Using High Resolution Experimental Data	Taehun Lee (City College of New York)

## NEUP-Related HTGR Projects – Universities Participated



## List of NEUP-Related HTGR Projects (FY09 – 23)



# Thermal-Fluid Phenomena and Operating/Accident Scenario - Studied in NEUP HTGR Projects

#### Thermal-Fluid Phenomena

- Plenum Mixing / Jet Impingement
  - Lower plenum
  - Upper plenum
  - Plenum to plenum
- Ingress
  - Air ingress
  - Steam/Water ingress
- Conjugate Heat Transfer
  - Forced convection
  - Natural convection
- RCCS Performance
- Core Bypass Flow
- Fluid Stratification
- Multi-physics (Fission product, safety analysis, thermal-mechanical etc.)

#### Various Scenarios

- Normal Operation
- Pressurized loss of forced cooling (PLOFC);
- Depressurized loss of forced cooling (DLOFC);
- Load Change (Transient)
- Steam Generator Accident (Tube break)

# Thermal-Fluid Phenomena and Accident Scenario – Database Matrix

			SCENARIOS								
	PROJECT # in RED ONGOING NEUP PROJECTS # in BLUE		NORMAL OPERATION	PRESSURIZED LOSS OF FLOW	DEPRESSURIZED LOSS OF FLOW	LOAD CHANGE (TRANSIENT)	STEAM GENERATOR TUBE BREAK				
	ING	LOWER PLENUM	12-3582 15-8627	16-10244	16-10244						
NA N	PLENUM MIXING ET IMPINGEMENT	UPPER PLENUM	12-3759	18-15058	18-15058						
PHENOMENA	PLE / JET	PLENUM TO PLENUM	13-4953 16-10509 23-29598	20-20074	20-20074	23-29598					
PH	INGRESS	AIR INGRESS		15-8205 20-19896	09-784 15-8205 13-4884 17-13115 14-6435 20-19896		14-6786 19-17183				
	INGI	STEAM/WATER INGRESS		18-15058	18-15058 21-24111		14-6786				

# Thermal-Fluid Phenomena and Accident Scenario – Database Matrix (Cont'd)

			SCENARIOS							
	PROJECT # in RED ONGOING NEUP PROJECTS # in BLUE		NORMAL OPERATION	PRESSURIZED LOSS OF FLOW	DEPRESSURIZED LOSS OF FLOW	LOAD CHANGE (TRANSIENT)	STEAM GENERATOR TUBE BREAK			
/	CONJUGATE HEAT TRANSFER (CORE)	FORCED CONVECTION	09-771 21-24104 11-3081 21-24287 16-10509	11-3218 21-24104 21-24287	11-3218 21-24156					
	CONJU HEAT TF (CO	NATURAL CONVECTION	14-6794 16-10509	14-6794	14-6435 21-24156					
PHENOMENA	R	CCS PERFORMANCE	09-781 09-817 11-3079 13-4953 20-19896	20-19896	20-19896	09-781 11-3079 09-817 13-5000				
PHEN	(	CORE BYPASS FLOW	09-830 15-8627 20-19968	15-8205	09-840 15-8205					
	FLUID STRATIFICATION				14-6435					
	MULTI-PHYSICS (FISSION PRODUCT, SAFETY ANALYSIS, THERMAL-MECHANICAL, ETC.)		20-19896 22-26607	20-19896	19-17037 21-24111 20-19896 21-24156	22-26607				

			SCENARIOS					
	DISHED NEUP DIECT # in RED	ONGOING NEUP PROJECTS # in BLUE	NORMAL OPERATION	PRESSURIZED LOSS OF FLOW	DEPRESSURIZED LOSS OF FLOW	LOAD CHANGE (TRANSIENT)	STEAM GENERATOR TUBE BREAK	
	ING	LOWER PLENUM	12-3582 15-8627	16-10244	16-10244			
	PLENUM MIXING / JET IMPINGEMENT	UPPER PLENUM	12-3759	18-15058	18-15058			
	PLE]	PLENUM TO PLENUM	13-4953 16-10509 23-29598	20-20074	20-20074	23-29598		
	GATE ANSFER INGRESS RE)	AIR INGRESS		15-8205 20-19896	09-784 15-8205 13-4884 17-13115 14-6435 20-19896		14-6786 19-17183	
Y.		STEAM/WATER INGRESS		18-15058	18-15058 21-24111		14-6786	
PHENOMENA		FORCED CONVECTION	09-771 21-24104 11-3081 21-24287 16-10509	11-3218 21-24104 21-24287	11-3218 21-24156			
Hd	CONJUGATE HEAT TRANSFER (CORE)	NATURAL CONVECTION	14-6794 16-10509	14-6794	14-6435 21-24156			
	RCCS PERFORMANCE		09-781 09-817 11-3079 11-3079	20-19896	20-19896	09-781 11-3079 09-817 13-5000		
	CORE BYPASS FLOW		09-830 15-8627 20-19968	15-8205	09-840 15-8205			
	FLU	JID STRATIFICATION			14-6435			
	MULTI-PHYSICS (FISSION PRODUCT, SAFETY ANALYSIS, THERMAL-MECHANICAL, ETC.)		20-19896 22-26607	20-19896	19-17037 21-24111 20-19896 21-24156	22-26607	<	

#### **Urgent Need for NEUP-HTGR Data Platform**

- NEUP projects have produced valuable results with both computational and experimental studies, but detailed information is not always available, including:
  - Detailed facility description;
  - Instrumentation locations;
  - Boundary conditions;
  - Resulting experimental data (raw/processed), etc.
- It is crucial and urgent to construct a more transparent, sustainable, and equitable process for the community to harvest important HTGR thermal fluid validation data created with U.S. taxpayer funding.
  - ART-GCR program will create a central data platform at INL to <u>identify</u>, <u>organize and</u>
     <u>store</u> the results of these valuable research projects, and
  - provide future guidance for storage and transmission of important project documentations for later NEUPs.

## Online Data Platform for NEUP HTGR (in progress)

DOE-ART NEUP Website NEUP Prj Files

NDMAS

#### NEUP LIBRARY

- Organized by FY and NEUP project number.
- Each entry currently has its resultant scientific publications:
  - Project abstract
  - Final report
  - Journal publications
  - Conference proceedings, etc.
- Methods and possibilities under investigation to include experimental data matrices.
- This will be integrated into the ART-GCR official webpage later.

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	Name	Document Title	PI/Authors	Doc Type	Awarded University (PI affiliated)	Year	Project ID / Title
roject ID	/ Title : 23-30994-IRF	P / Exascale Simulation of Thermal-Hydraulics F	henomena in Advanced Rea	ctors and Val	idation Using High Resolution Exp	periment	tal Data (1)
edf - E3-	IRP 23-30994 *	Exascale Simulation of Thermal-Hydraulics Phenomena in Advanced Reactors and Validation Using High Resolution Experimental Data	Taehun Lee	Abstract	City College of New York	2023	23-30994-IRP / Exascale Simulation of Thermal-Hydraulics Phenomena in Advanced Reactors and Validation Using High Resolution Experimental Data
roject ID	/ Title : 23-29598 / U	ncertainty Quantification of Model Extrapolation	in Neural Networkinformed	Turbulent Cle	osures for Plenum Mixing in HTGF	Rs (1)	
idf -D-	23- 29598_Technical Abstract #	Uncertainty Quantification of Model Extrapolation in Neural Networkinformed Turbulent Closures for Plenum Mixing in HTGRs	Som Dutta	Abstract	Utah State University	2023	23-29598 / Uncertainty Quantification of Model Extrapolation in Neural Networkinformed Turbulent Closures for Plenum Mixing in HTGRs
roject ID	/ Title : 22-26607 / A	n Innovative Monitoring Technology for the Rea	ctor Vessel of Micro-HTGR (	1)			
pdf - Co-	22-26607 Technical Abstract	An Innovative Monitoring Technology for the Reactor Vessel of Micro-HTGR	Lesley Wright	Abstract	Texas A&M Engineering Experiment Station	2022	22-26607 / An Innovative Monitoring Technology for the Reactor Vessel of Micro- HTGR
roiect ID	/ Title : 21-24287 / In	vestigating heat transfer in horizontal micro-H	GRs under normal and PCC	conditions (2	)		
idf .	21-24287 Technical Abstract	Investigating heat transfer in horizontal micro-HTGRs under normal and PCC conditions	Hitesh Bindra	Abstract	Kansas State University	2021	21-24287 / Investigating heat transfer in horizontal micro-HTGRs under normal and PCC conditions
pdf -CI-	Ross et al. 2023	Passive heat removal in horizontally oriented micro-HTGRs	Molly Ross, T-Ying Lin, Isaiah Wicoff, Broderick Sieh, Piyush Sabharwall, Donald McEligot, Hitesh Bindra	Article	Kansas State University	2023	21-24287 / Investigating heat transfer in horizontal micro-HTGRs under normal and PCC conditions
roiect ID	/ Title : 21-24156 / Ex	operimental thermofluidic validation of TCR fuel	elements using distributed t	temperature a	and flow sensing (1)		
edf +D=	21-24156 Technical Abstract	Experimental thermofluidic validation of TCR fuel elements using distributed temperature and flow sensing	Hitesh Bindra	Abstract		2021	21-24156 / Experimental thermofluidic validation of TCR fuel elements using distributed temperature and flow sensing
roject ID	/ Title : 21-24111 / Ex	operimental Investigations of HTGR Fission Pro-	duct Transport in Separate-e	ffect Test Fac	cilities Under Prototypical Condition	ons for E	Depressurization and Water-ingress Accidents (
ing the second	21-24111 Technical Abstract	Experimental Investigations of HTGR Fission Product Transport in Separate-effect Test Facilities Under Prototypical Conditions for Depressurization and Water-ingress Accidents	N.K. Anand	Abstract	Texas A&M University	2021	21-24111 / Experimental Investigations of HTGR Fission Product Transport in Separate- effect Test Facilities Under Prototypical Conditions for Depressurization and Water- ingress Accidents
roject ID	/ Title : 21-24104 / TI	nermal Hydraulics Investigation of Horizontally	Orientated Layout Micro HTG	Rsunder Nor	mal Operation and PCC Condition	s Using	Integrated Advanced Measurement Techniques
edf C	21-24104 Technical Abstract	Thermal Hydraulics Investigation of Horizontally Orientated Layout Micro HTGRsunder Normal Operation and PCC Conditions Using Integrated Advanced	Muthanna H. Al-Dahhan	Abstract	Missouri University of Science & Technology	2021	21-24104 / Thermal Hydraulics Investigation of Horizontally Orientated Layout Micro HTGRsunder Normal Operation and PCC Conditions Using Integrated Advanced

#### Ongoing V&V Efforts

#### - Ranking Table Development for Experimental Facilities

- Incorporated the HTGR-related projects:
  - NEUP projects
  - Available experimental facilities, such as ANL NSTF, OSU HTTF, etc.
- Identified investigation topics and phenomena of interest.
- Collected information for data resolution and time scheme.
- Ranking parameters:
  - Raw data availability;
  - Data uncertainties;
  - Scientific publications;
  - Computational models;
  - Integral vs. Separate effect test.

## Ranking Table Development for HTGR-related Experimental Facilities

	·						•							
Facility/ NEUP Project	Location	Point of Contact (PI)	Phenomena of Interest / Purpose	Data Resolution (Description)	Time scheme: [SS, Transient]	Topic Type: [TH, Neutronics, Chemical, Multi-physics]	Raw Data (#)	Data Uncertainties (#)	Report/ Publications (#)	Computational Models (#)	Integral? (#)	Score		
NEUP 17-13115	Univ. of Michigan	Victor Petrov	He/Air mixing behavior during the HTGR DLOFC and Air Ingress accidents for small and medium sized breaks.	Jet velocity data (PIV and LDV), mass flow rate, temperature, pressure	Transient	ТН	80	70	60	70	70	350		
NEUP 11-3081	Utah State University	Barton Smith	PIV vs CFD data validation for a heated flat plate	Heat flux, Velocity field (PIV)	SS	TH	80	70	50	60	80	340		
NEUP 12-3759	TAMU	Yassin Hassan	(Jet) Flow mixing at upper/lower plenum	Velocity field (PIV), Temperature	SS	тн	80	70	60	60	70	340		
NEUP 18-15058	Univ. of Michigan	Xiaodong Sun	High-resolution data on jet interaction at HTGR upper plenum; Moisture absorption of heated graphite.	Velocity field (PIV and LDV)	SS, Transient	TH, Chemical	70	70	60	70	70	340		
NEUP 15-8627	Univ. of Pittsburgh	Mark Kimber	Flow interaction between core bypass and coolant jet flows	Velocity Field (PIV), Reynolds Stress, Turbulent kinetic energy, vorticity	SS	TH	70	70	70	60	70	340		
NEUP 13-5000	Univ. of Michigan		RCCS at upper plenum and turbulent jet mixing	Velocity field (PIV and LDV)	SS	TH	80	70	50	60	70	330		
NEUP 09-817	TAMU	Yassin Hassan	RCCS in VHTR	Temperature (k-type TC) and mass flow rate (flow meter)	Transient	ТН	70	60	60	60	60	310		
NEUP 09-781	UW, Madison	Michael Corradini	Water RCCS performance	Temperature and mass flow rate (time resolved)	SS, Transient	TH	70	60	70	60	50	310		
NEUP 16-10509	Utah State University	Barton Smith	Buoyancy driven/opposed flow measurements	Velocity field (PIV), temperature (TC), pressure, mass flow rate	SS, Transient	TH	70	70	50	50	70	310		
NEUP 12-3582	Univ. of Pittsburgh	Mark Kimber	Turbulent jet mixing in the lower plenum	Velocity field (PIV), temperature (TC, IR camera for field)	SS	ТН	70	70	50	60	60	310		
Natural Convection Shutdown Heat Removal Test Facility (NSTF)	ANL	Darius Lisowski	RCCS performance using both water and air	Mass flow rate, temperature, pressure	SS, Transient	ТН	70	60	70	70	30	300		

(to be continued).....

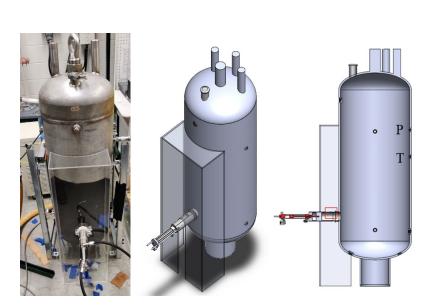
# ART-NEUP Collaborations with INL LDRD project

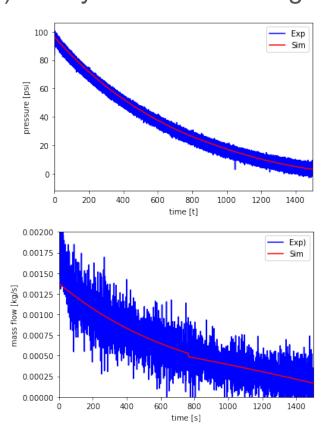
Project Title: A Causal Approach to Model Validation and Calibration

Proceedings of the ASME 2023 International Mechanical Engineering Congress and Exposition October 29-November 2, 2023, New Orleans, Louisiana

IMECE2023-112430

- Lead PI: Dr. Diego Mandelli
- Selected collaboration project: NEUP 17-13115 Helium Air Ingress gas Reactor Experimental (HAIRE) facility at Univ. of Michigan.





#### A CAUSAL APPROACH TO MODEL VALIDATION AND CALIBRATION

D Mandelli R. Gonzales C. Wang M. Abdo Z. Welker Idaho National Idaho National Idaho National University of Laboratory Laboratory Laboratory Michigan Laboratory Idaho Falls, ID Idaho Falls. ID Idaho Falls, ID Idaho Falls, ID Ann Arbor, MI

> P. Balestra S. Qin Idaho National Idaho National Laboratory Laboratory Idaho Falls, ID

V. Petrov University of Michigan Ann Arbor, MI

A. Manera University of Michigan Ann Arbor, MI

#### ABSTRACT

All current developed methods for model validation are based on standard statistical analysis (to measure statistical differences between data populations) or machine learning (to identify response surface from data) methods. Both classes of methods have in common to be purely data driven, i.e., they provide quantitative comparison measures between data sets (e.g., simulated vs. measured data) without explicitly considering the hypothesis behind them (e.g., boundary conditions) and the structure of the employed models. This generates sometimes the erroneous conclusion that when two data populations are "close enough" then the models that have generated them are "similar". In addition, when simulated and experimental data differs beyond the acceptance criteria, model calibration techniques are used to "tweak" simulation model parameters to reduce the gap between simulated and experimental data. This gives the false expectation that a simulation model matches reality. The goal of this paper is to move away from currently employed purely data-driven methods for validation and calibration toward more robust model-driven methods based on causal inference. The presented methods are designed to capture the causal relationships between data elements (e.g., simulated and experimental data) rather than looking at their associations, and they employ these relationships to measure differences between simulated and measured data. These causal differences then directly inform the calibration process rather than relying on the analyst educated

Keywords: validation, causal inference, calibration

#### 1. INTRODUCTION

Currently, advanced simulation models are not only used in the design and licensing of systems, but are also an integral part of their operation and control (e.g., in system autonomous operation or in a digital twin context). These simulation models are designed to mimic aspects of the real world as closely as possible to correctly capture the evolution and the properties of those aspects. Creation of these simulation models begins by defining their constituent laws, as represented through a set of mathematical equations (e.g., conservation laws, equations of motion, thermodynamic laws, etc.). Note that, at this stage, the causal relationships between variables defined within the model are totally lost due to the nature of the mathematical formalism being used. Causal relationships are regained when the developed mathematical equations are coded into a simulation

Once the coding process is completed and verified, these models require a validation process that tests whether they can model aspects of the real world, by comparing simulated results with experimental real-world observations under similar initial/boundary conditions. Current methods for model validation are based on standard statistical analysis (to measure statistical differences between data populations) or machine learning (ML) (to identify response surfaces from data) methods. Both classes of methods are purely data driven, meaning that they provide quantitative comparison measures between datasets (e.g., simulated vs. measured data) without explicitly considering the hypotheses behind them (e.g., boundary conditions) or the structure of the employed models. This can lead to the erroneous conclusion that, if two data populations are close enough, then the models that generated them are similar. In

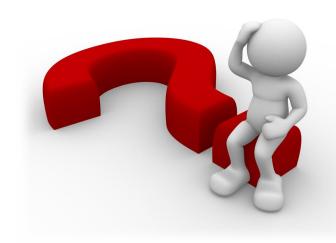
#### Conclusion

- Updated the matrix for Thermal-Fluid Phenomena and Accident Scenario with new projects.
- Communicating with several awarded universities and gathering experimental data as well as computational models.
- Collaborating with INL research team through LDRD project
  - Ranking table of code V&V for experimental facility.
- Online data platform in development.

#### **Future Work**

- Communicating with university PIs to gather more detailed information for experimental and computational work.
  - Refining the HTGR phenomena summary chart continuously.
- Finalizing the data platform and keeping it updated with new funded NEUP-HTGR projects.
- ➤ Choosing 1-2 projects and performing benchmark studies during next fiscal year.

# Thank you for your attention! Questions?





#### Reference

- 1. U.S. Department of Energy, Nuclear Energy University Program (NEUP). [cited 2023 July]; Available from: <a href="https://neup.inl.gov/SitePages/Home.aspx">https://neup.inl.gov/SitePages/Home.aspx</a>.
- 2. INL Advanced Reactor Technologies (ART) Program. [cited 2023 July]; Available from: <a href="https://art.inl.gov/SitePages/ART%20Program.aspx">https://art.inl.gov/SitePages/ART%20Program.aspx</a>.
- 3. Qin, S., Song, M., Vietz, S. H., T Pham, C. B., Plummer, M. A., & Strydom, G. (2022). *High-temperature gas-cooled reactor research Survey and Overview: Preliminary data Platform construction for the nuclear energy university Program* (No. INL/RPT-22-68771-Rev000). Idaho National Lab.(INL), Idaho Falls, ID (United States). Available from: <a href="https://doi.org/10.2172/1887092">https://doi.org/10.2172/1887092</a>.