



NDMAS: Data storage, visualization, analysis, delivery, and more

July 2023

Changing the World's Energy Future

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NDMAS

Data storage, visualization, analysis, delivery, and more

DOE ART Gas-Cooled Reactor (GCR) Review Meeting

Virtual Meeting

July 25 – 27, 2023



Motivation

- As scientists, we have a responsibility and a desire to properly preserve our very expensive, publicly funded data
- DOE Public Access Plan
- Users might include:
 - Reactor vendor engineers
 - NRC
 - Researchers
 - Universities





NDMAS Data

- Advanced Reactor Technologies
 - Fuel Fabrication
 - Irradiation Monitoring (Fuel & Graphite – near real-time)
 - Post-Irradiation Examination (Fuel & Graphite)
 - Graphite Characterization
 - High Temperature Materials Mechanical Tests
 - Methods Validation Data
 - JAEA's High Temperature Test Reactor
 - Argonne National Laboratory's Natural convection Shutdown heat removal Test Facility (NSTF)
 - Oregon State University's High Temperature Test Facility (HTTF)
- Other program data
 - Advanced Test Reactor operations (near real-time)
 - Accident Tolerant Fuel
 - TMIST (near real-time)
 - High Performance Research Reactor
 - European Mini-plate Irradiation Experiment (EMPIRE)
 - Advanced Fuel Campaign



Nuclear Data Management and Analysis System (NDMAS)

- Provide a controlled and central repository and data management system for R&D data
- Provide qualification traceability to level required in support of design decisions and licensing
- Provide extensive statistical analysis and graphing capabilities
- Enable access-controlled web delivery of data, graphs, and analysis results to the research community
- Provide web-based collaboration capabilities to the research community
- Provide an adaptive system to meet individual program needs



NDMAS Data Processing Model

Data Owner



Capture

- Programmed data capture from files
- Multiple checks on process integrity

Archive

- Original data file, codes, and completed QA forms are retained
- Stored on the servers

Storage

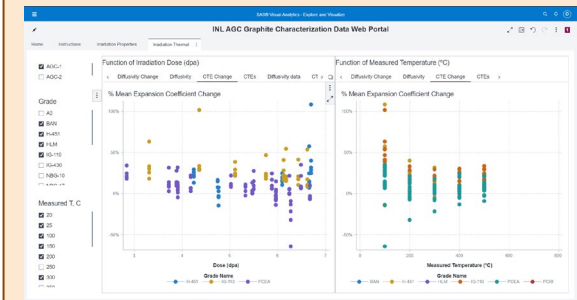
- Secure file storage using robust servers
- Data storage in relational database
- Version control of codes, documents, and source data

Qualification

- Meet NQA-1 data requirements
- Ensure data integrity from capture to delivery
- Process tracking of code & webpage changes

Delivery & Analysis

- SharePoint and SAS VA web interface
- Access controlled
- Data tables (downloadable)
- Graphical displays
- Advanced statistical analyses



End User





Data Storage

NDMAS 2.0 Database

- Data that need to be readily accessible for analysis and dissemination are stored in the database
- Structured Query Language relational database
 - Uses Microsoft SQL Server technology
 - Database structure (schema) is customized for each project
 - Efficient and customizable visualization and reporting
- Access
 - Access to database structure controlled to NDMAS team only
 - Access to data within database controlled with approved authentication
- Quality
 - Conforms to INL Information Management standards
 - Operates within framework of NQA-1 2008 / NQA-1a-2009 Addenda

Reference: SDD-228 Rev. 1, “NDMAS 2.0 Database,” March, 2015.

Web Delivery

- Uses SharePoint with SAS VA to deliver experimental results to the program community over the Internet
 - Downloadable data tables
 - Graphs
 - Analysis results
- Full access control
 - In INL's DMZ
 - Role-based
 - Sites
 - Subsites
 - Row-level data security

The screenshot displays a SharePoint web page for the NDMAS (Nuclear Data Management and Analysis System) High Temperature Reactor Program. The page features a blue header with the SharePoint logo and navigation links. The main content area includes a title 'High Temperature Reactor Program' and a detailed description of HTGRs. A diagram on the right illustrates the reactor's role in producing hydrogen and electricity. The footer contains contact information for the ART Program and NDMAS, along with INL logos and a copyright notice for 2014 Idaho National Laboratory.

SharePoint

NDMAS

High Temperature Reactor Program

High temperature gas-cooled reactors (HTGRs) are graphite-moderated nuclear reactors cooled by helium. The high-outlet temperatures and high thermal-energy conversion efficiency of HTGRs enable efficient and cost-effective integration with non-electricity generating applications, such as process heat and hydrogen production, for the many petrochemical and other industrial processes that require operating temperatures between 300°C and 900°C. Using HTGRs in this way would supplement the use of premium fossil fuels, such as oil and natural gas, improve overall energy security in the United States by reducing dependence on foreign fuels, and reduce CO₂ emissions. Key characteristics of this reactor design are the use of helium as a coolant, graphite as the moderator of neutrons, and ceramic-coated particle fuel. Helium is chemically inert and neutronically transparent. The graphite core slows down the neutrons, provides high-temperature strength and structural stability for the core, and acts as a substantial heat sink during transient conditions. The ceramic-coated particle fuel is extremely robust and retains the radioactive byproducts of the fission reaction under both normal and off-normal conditions.

This site contains data from the High Temperature Reactor (HTR) Program. Data from the program are organized around three program areas. These are Fuel Development and Qualification, Graphite Technology Development, and High-Temperature Materials Development. A key component of the HTR program is determining the behavior of fuel and graphite under irradiation. To accomplish this, the HTR program is running two experimental series. Fuel irradiation behavior is being determined with the Advanced Graphite Reactor experiments (AGR). Graphite irradiation behavior is being investigated through the Advanced Graphite Creep experiments (AGC).

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Fuel Fabrication

SharePoint

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BROWSE PAGE PUBLISH



ATR Ops EMPIRE HPRR HTR Methods T-MIST NDMASTeam EDIT LINKS

Nuclear Data Management and Analysis System > High Temperature Reactor > Fuel Development and Qualification > Fuel Fabrication

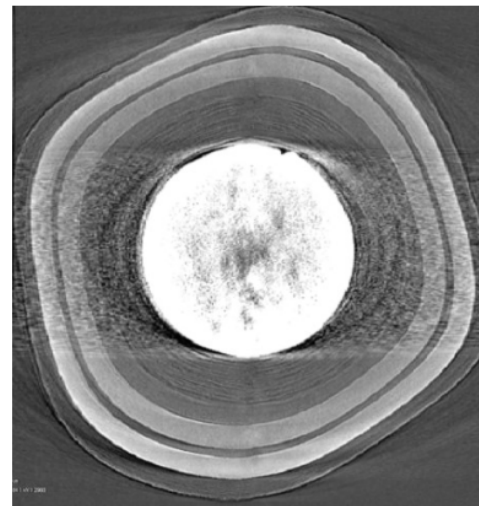
Fuel Fabrication

The Fuel Fabrication project develops and qualifies fuel fabrication processes that can serve as the foundation for fabrication of commercial-scale, coated-particle fuel for HTGRs. The fabrication process developed within the VHTR TDO/AGR fuel program begins with UCO or UO₂ kernels formed by the internal gelation process in which droplets of uranium-containing chemical broth are formed into gel spheres in a fluid medium. The resulting gel spheres are then dried and sintered into hard ceramic spheres yielding kernels of a controlled and consistent size and chemistry.

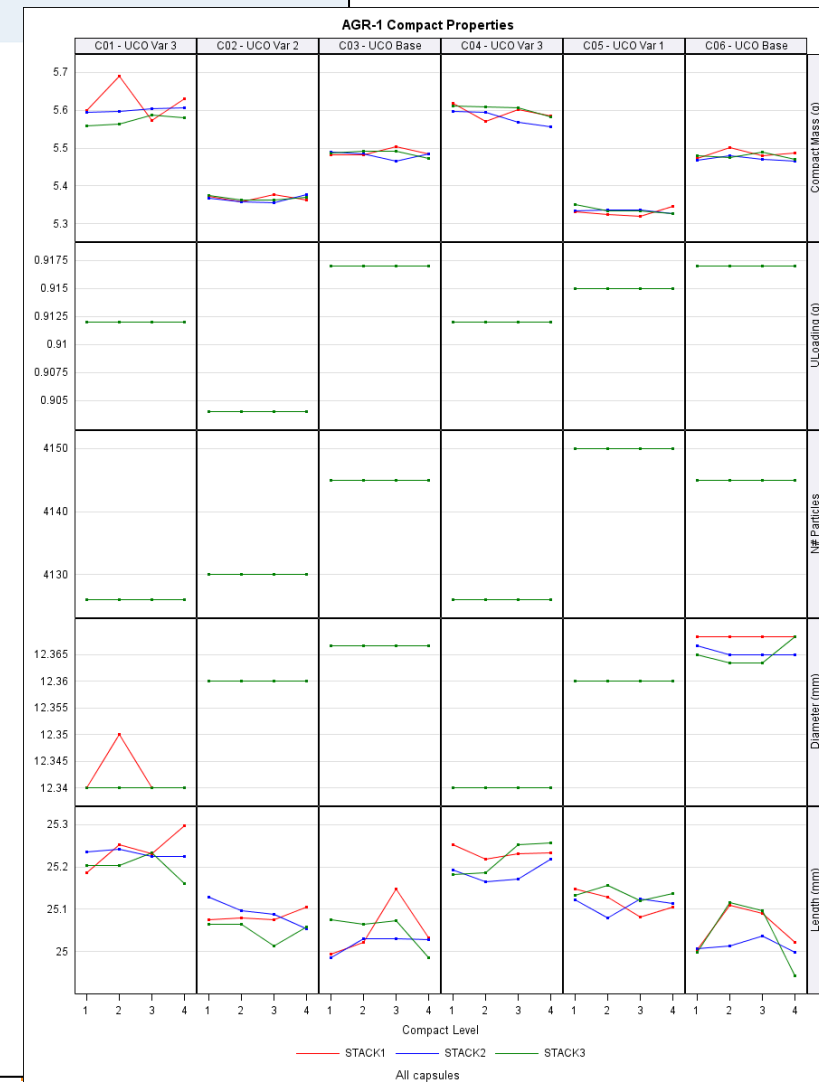
Fuel kernels are coated using a fluidized-bed chemical vapor deposition process. The coatings include a low-density carbon layer (buffer layer), a high-density inner pyrolytic carbon (IPyC) layer, a silicon carbide (SiC) layer, and a high-density outer pyrolytic carbon (OPyC) layer. These coatings are designed to work together to make each fuel particle a mini pressure vessel that will maintain its integrity and retain fission products during normal reactor operation and potential accident conditions. The finished coated particle is a small (~1 mm diameter) carbon and ceramic sphere that is stable to temperatures well beyond 1,600°C.

Fuel particles are embedded in a thermosetting matrix consisting of phenolic resin mixed with graphite powder and molded into right-circular cylinders. Each cylinder is called a compact. A compact contains from 1,800 to 4,200 particles depending on the uranium loading per kernel and the desired total uranium content per compact. Each AGR capsule contains only one fuel variant.

Data were received as PDF files on a CD and were manually transferred into an EXCEL spreadsheet. The data were then read into SAS datasets from the EXCEL spreadsheet. The referential integrity of the data was evaluated to make sure that all components, component attributes, response variables, and response values were properly linked. The SAS datasets were then transferred to the NDMAS SQL database for permanent storage. Data were then pulled from the SQL database and compared to the SAS dataset used to populate the database using a SAS procedure (PROC-COMPARE), verifying that the data in the SQL database are the same as the data in the SAS datasets. Finally, the data in the SQL database output were compared to the original data in the data packages by an independent person. For fabrication data, the delivered hard copy data packages are served as the quality record showing whether the predefined design specifications were met or have been accepted as is. Because all laboratories providing data are either NQA-1 compliant or were assessed to have QA programs that were equivalent to NQA-1.



Tomographic cross-section of a particle with extra outer layers

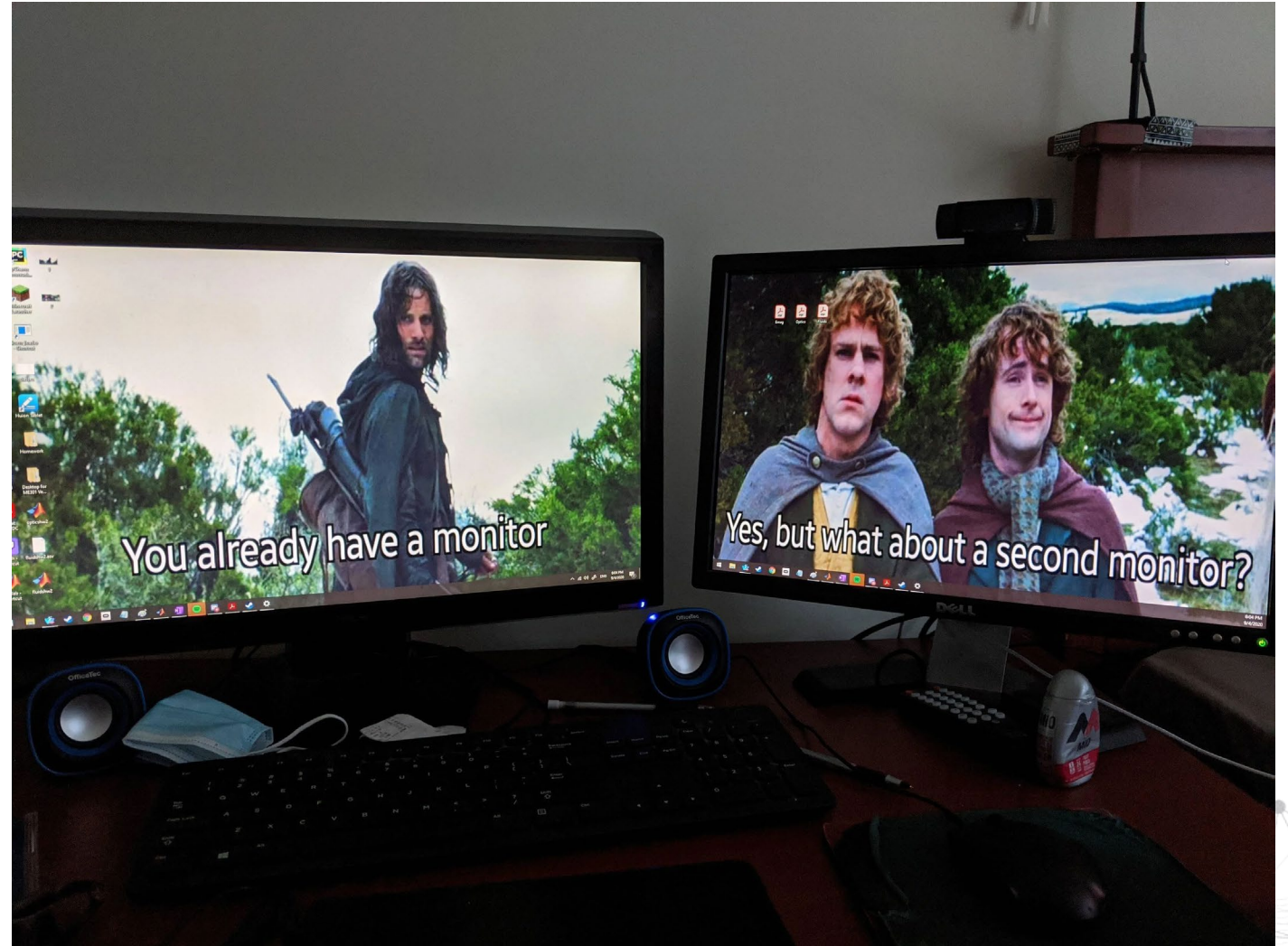


PIE

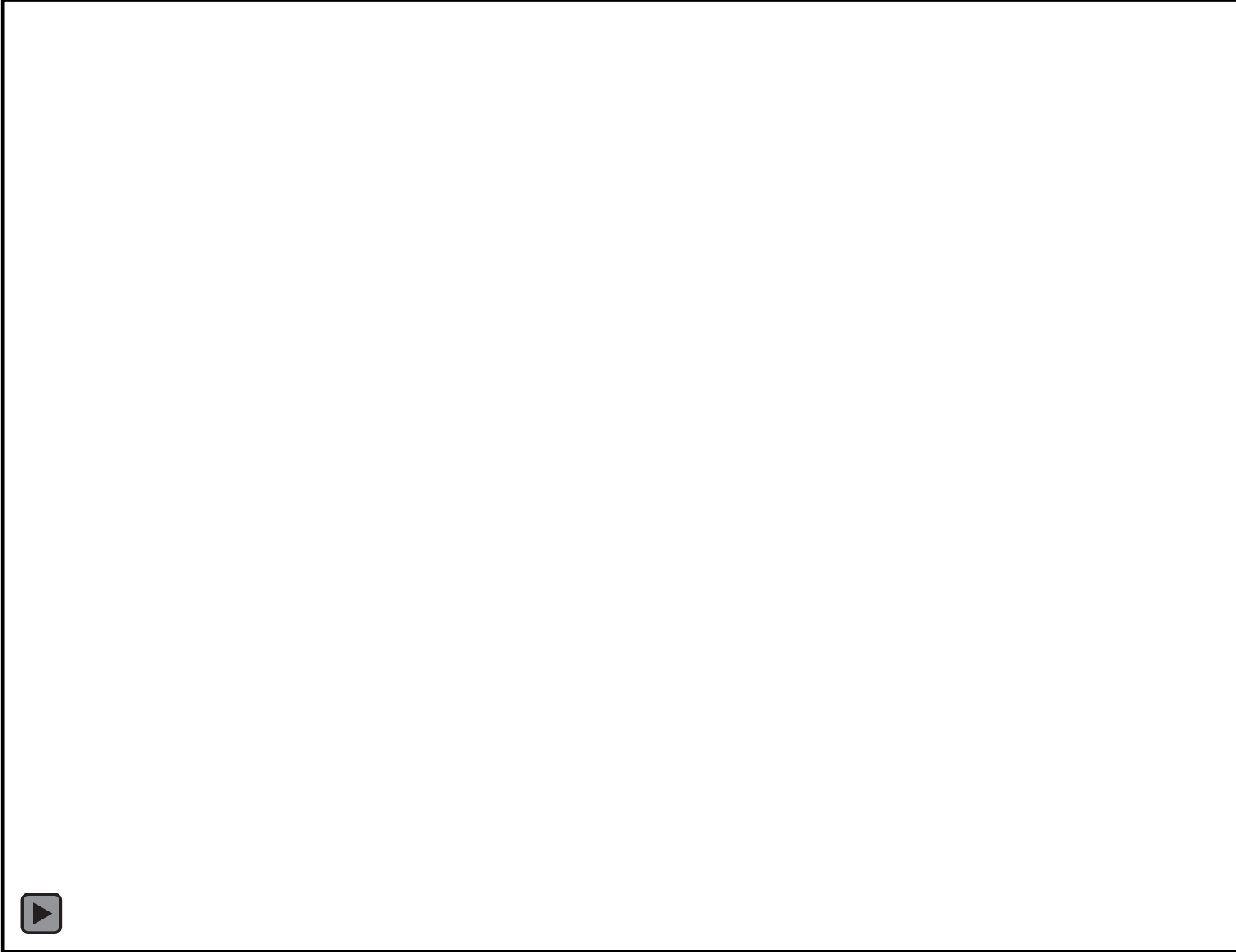


ATR Experiment Monitoring Capabilities

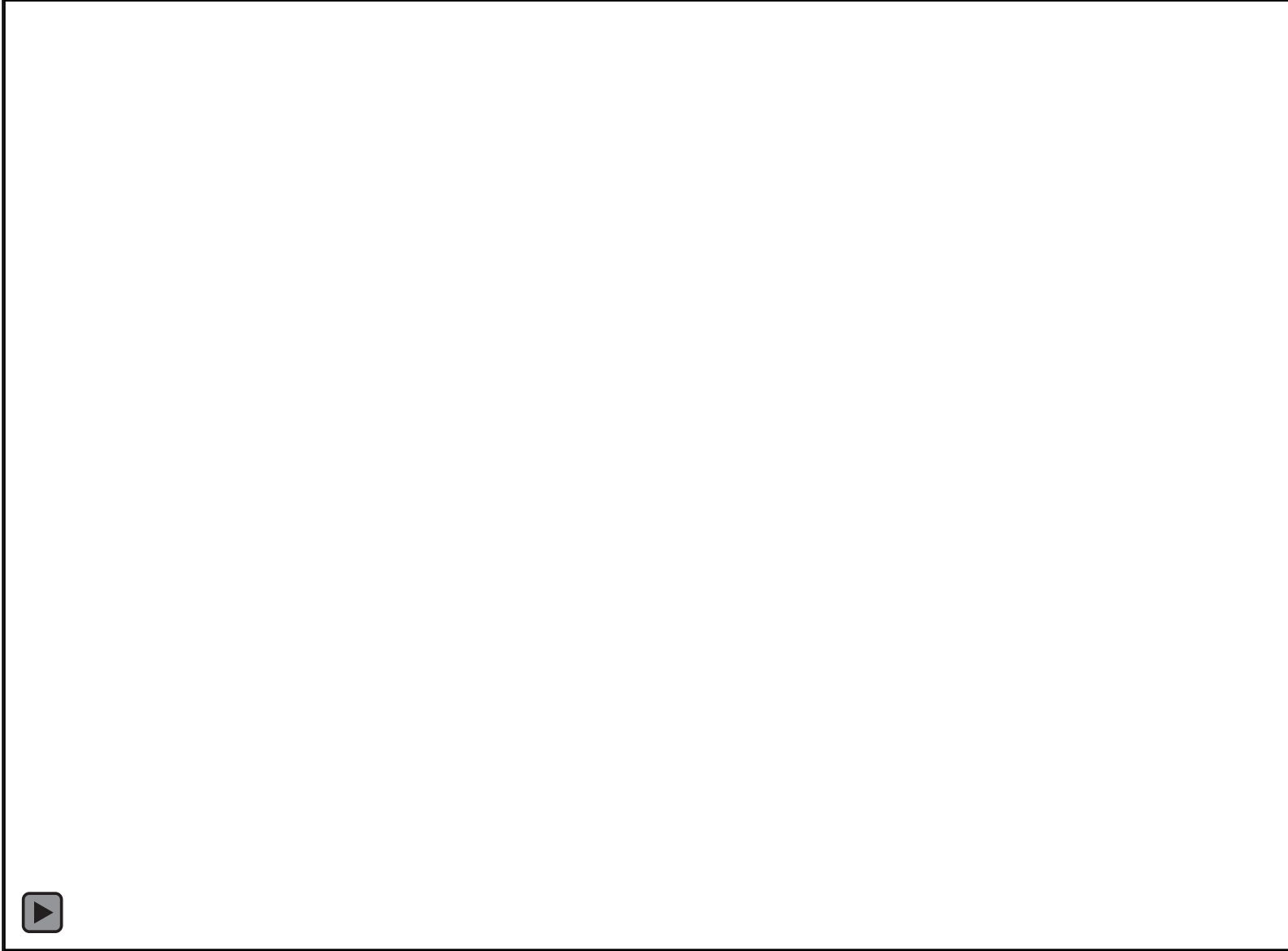
- Reactor power and experiment monitoring data that are delivered several times a day
- Data are processed and pushed to the database immediately
- Data and plots are updated in SAS at least once a day
- Researchers can view most recent plots or download data
- Excel files for data from completed experiments have been generated and stored in SharePoint libraries for easy access



ATR Operations



Advanced Gas Reactor Irradiation Monitoring



Advanced Gas Reactor Irradiation Monitoring







Documents

- All controlled documents are stored in EDMS
- SharePoint libraries can be used to store and organize important documents that need to be more accessible
 - As a file (pdf or word document)
 - As a link to OSTI or EDMS (only accessible within the firewall)
- Term Sets can be used to categorize documents if desired
- Custom search applications can be developed as needed
- All documents included would have to have a full LRS release
- Currently we only have select reports stored in SharePoint

Example: Fuel Irradiation Web Page

Data Qualification and Analysis Documents

The reports listed below are not record copies and may not represent the most recent version. The INL Electronic Document Management System (EDMS) should be used to provide the record copy of the document. The ECARs listed below have a direct link to EDMS. Unfortunately, these links will not work for users outside the INL firewall. External users who wish to obtain a copy of a document stored in EDMS should contact [Courtney Otani](#), 208-526-1228.

Name	Document ID
▀ Experiment : AGR-1 (9)	
AGR-1 Thermocouple Data Analysis	INL/EXT-12-24761
AGR-1 Final As-Run Report Rev3	INL/EXT-10-18097
AGR-1 JMOCUP As-Run Daily Depletion Calculation	ECAR-958
AGR-1 Daily As-Run Thermal Analysis	ECAR-968
Release-to-Birth Ratios for AGR-1 Operating Cycles 138B through 145A	ECAR-907
AGR-1 Data Qualification Report	INL/EXT-10-17943
Uncertainty Quantification of Calculated Temperatures for the AGR-1 Experiment	INL/EXT-12-25169
AGR-1 Irradiation Experiment Test Plan Rev 3	INL/EXT-05-00593
AGR-1 Test Specification Rev1	EDF-4731
▸ Experiment : AGR-2 (8)	
▸ Experiment : AGR-3/4 (10)	
▸ Experiment : AGR-5/6/7 (8)	

Working and Interfacing with NDMAS

- Working with NDMAS
 - Meet to discuss needs
 - NDMAS team members become members of your team
 - Database or tables are set up
 - Delivery is set up
 - Iterative process
 - Set up for next data set or stream will utilize established work
- New users
 - Request account by sending email to ndmas-webadmin@inl.gov
 - Need an INL POC to verify their need to access data
 - A restricted party screening is run
 - International employees require a security plan
 - They will be added to one or more SharePoint (and SAS if needed) security groups based on the data they need to access

Process for Completed Experiments

- As experiments close out, convert most data downloads to downloadable Excel files
- At end of project
 - Records retention 25 years past end of the project
 - Archive raw data files and the database to EDMS
 - As long as possible with minimal intervention:
 - Maintain the database
 - Maintain the web site
 - Add new users



FY 23 Accomplishments and Look Ahead

- All milestones completed on time
 - March- Presentation to HTR TWG
 - June- Updated thermal model for AGR-5/6/7 to analyze shifting holder position
 - July- Revised dimensional change analysis report with AGR-5/6/7 data
- New major project set up
 - Gen IV Materials Handbook learning and transition to INL initiated in tight collaboration with Oak Ridge
 - Primary execution to begin in FY 24
- Improving delivery system
 - Updating delivery system to increase capabilities
 - Training with vendor to maximize use of tool
- Formalizing access process for AGR data
 - Updating so DOE and AGR program access requirements are met