

# DOE ART Graphite Program Intro Status 2023

July 2023

William E Windes





#### DISCLAIMER

This information was prepared as an account of work sponsored by an agency of the U.S. Government. Neither the U.S. Government nor any agency thereof, nor any of their employees, makes any warranty, expressed or implied, or assumes any legal liability or responsibility for the accuracy, completeness, or usefulness, of any information, apparatus, product, or process disclosed, or represents that its use would not infringe privately owned rights. References herein to any specific commercial product, process, or service by trade name, trade mark, manufacturer, or otherwise, does not necessarily constitute or imply its endorsement, recommendation, or favoring by the U.S. Government or any agency thereof. The views and opinions of authors expressed herein do not necessarily state or reflect those of the U.S. Government or any agency thereof.

#### **DOE ART Graphite Program Intro Status 2023**

William E Windes

**July 2023** 

Idaho National Laboratory Idaho Falls, Idaho 83415

http://www.inl.gov

Prepared for the U.S. Department of Energy Under DOE Idaho Operations Office Contract DE-AC07-05ID14517 July 26, 2023

#### **Will Windes**

ART Graphite Technical Lead Idaho National Laboratory

# **DOE ART Graphite Program**

Status 2023

DOE ART Gas-Cooled Reactor (CR) Review Meeting

Virtual Meeting
July 25 – 27, 2023



# Graphite Program Contributors to this discussion

Researcher	Expertise
Andrea L. Mack andrea.mack@inl.gov	ASME Code
Anne Campbell campbellaa@ornl.gov	PIE, Irradiation damage, Irradiation behavior
Arvin Cunningham arvin.cunningham@inl.gov	Oxidation, Split-disk testing
Austin C. Matthews austin.matthews@inl.gov	Material property testing, PIE, Oxidation
David T. Rohrbaugh david.rohrbaugh@inl.gov	Unirradiated and Irradiated material properties
Jose' D. Arregui-Mena arreguimenjd@ornl.gov	Microstructure, irradiation damage
Lu Cai Lu.Cai@inl.gov	Pebble Oxidation
Martin Metcalfe martin.p.metcalfe@gmail.com	HTR operations, ASME, ASTM

Researcher	Expertise			
Mary Kaye Aimes marykaye.ames@inl.gov	Oxidation, Material testing			
Michael E. Davenport michael.davenport@inl.gov	Irradiation experiments			
Nidia C. Gallego gallegonc@ornl.gov	Molten salt technical lead, irradiation damage			
Philip L. Winston philip.winston@inl.gov	Irradiation experiments			
Rebecca E. Smith rebecca.smith@inl.gov	Graphite oxidation (irr. and unirr)			
Steve Johns Steve.johns@inl .gov	Irradiation damage, Characterization, Split-disk			
William Windes william.windes@inl.gov	Irradiation behavior, ASME			
Wilna Geringer geringerjw@ornl.gov	ASME, Composites, Graphite			

#### Five different research areas

#### **Behavior models**

- Predicts irradiated material properties and potential degradation issues
- Irradiation behavior for continued safe operation

#### **Licensing & Code**

- Establishes an ASME approved code (for 1st time)
- Develops property values for initial components and irradiation induced changes

# Graphite R&D Program

Defines the safe working envelope for nuclear graphite and protection of fuel

### As-Fab'd Properties

- (Statistically) Establishes asreceived material properties
- Baseline data used to determine irradiation material properties

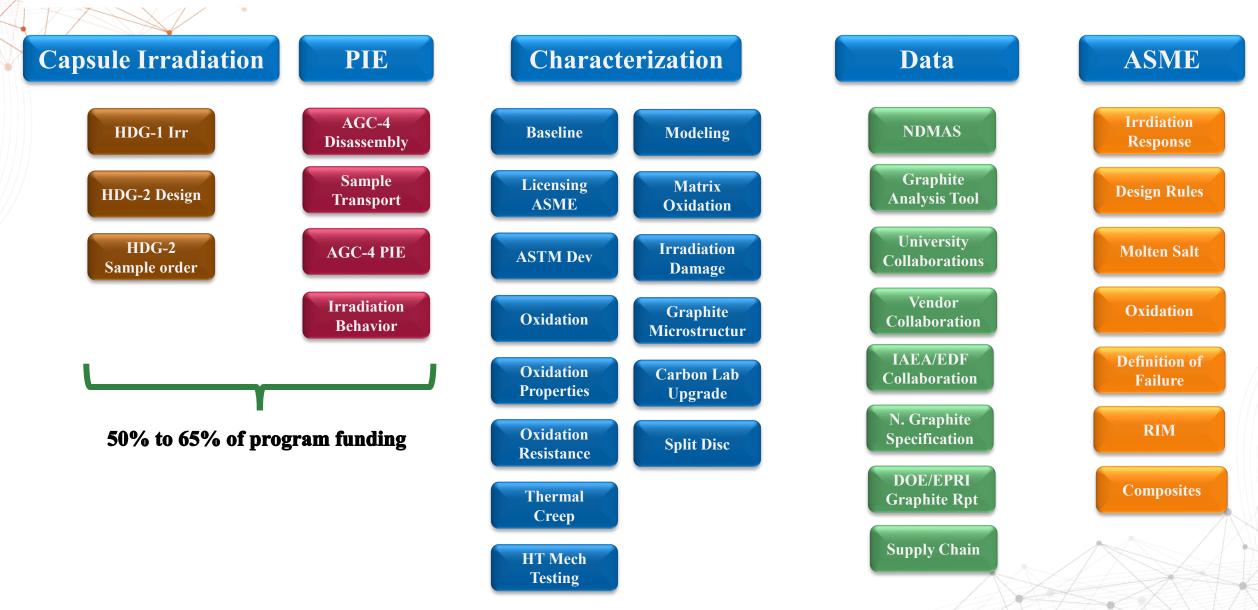
#### **Mechanisms and Analysis**

- Data analysis and interpretation
- Understanding the damage mechanisms is key to interpreting data

#### **Irradiation**

- Determines irradiation changes to material properties
- Irradiation behavior for continued safe operation

### **FY23 Graphite Activities**



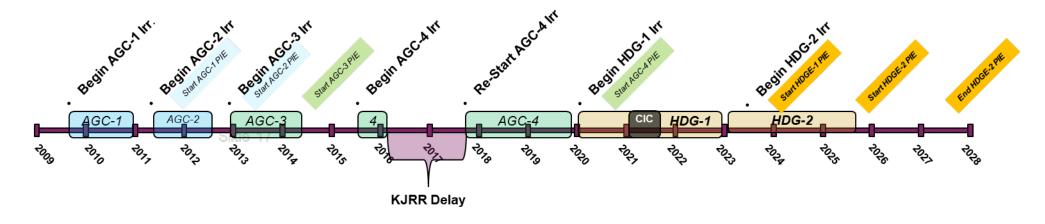
### What are we discussing this year?

- AGC Update
- Oxidation Activities
  - Oxidation rate, penetration/lathing, strength after oxidation
- Baseline Status
  - Data use in ASME and improved split-disc studies
- ASME Code Development:
  - Irradiation model
  - Ceramic Composites
  - Design task group
  - Component failure
- VIC Project Update
- Molten salt studies
  - We are initiating material interactions now

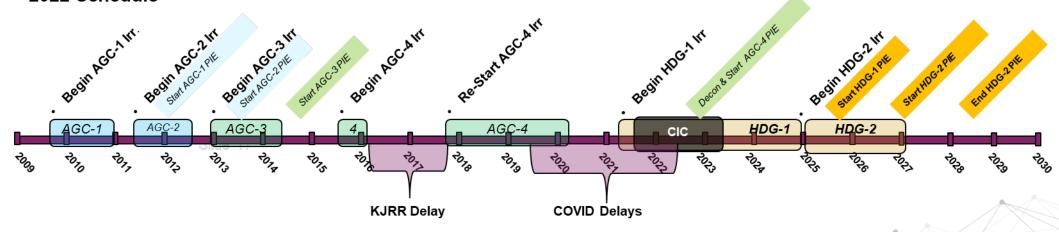
# Advanced Graphite Creep (AGC) Experiment (Will Windes)



#### 2018 Schedule



#### 2022 Schedule



### Oxidation Studies (Rebecca Smith)

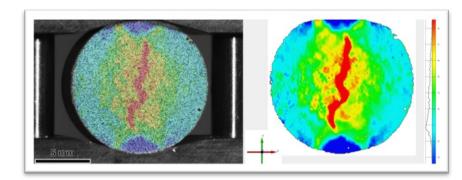
- Large amount of work being performed
  - Oxidation rate work is minimal
  - Mostly ASME and ASTM involved work
    - Strength after oxidation
    - Oxygen penetration depth
    - Oxidation of MS grades
- Oxidation of irradiated graphite

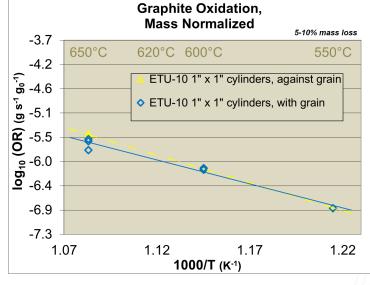


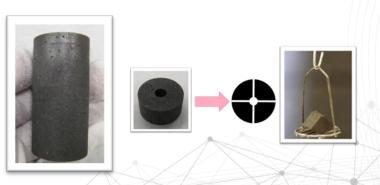










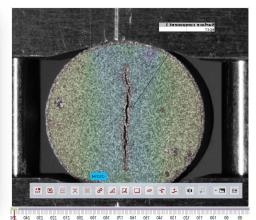


#### Baseline (Arvin Cunningham)



- Over 23,800 NQA-1 qualified measurements thus far
- 5 major graphite grades
- Focus is supporting other areas:
  - Irradiation: AGC
  - ASTM: Split-disc testing (D8289)
  - ASTM: High Temperature tests
  - ASME code rule development
  - Molten salt interactions

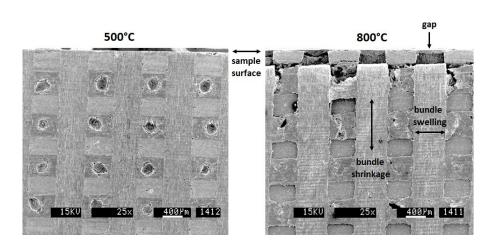




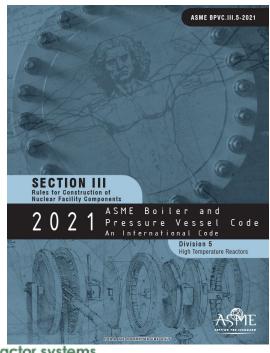
		Billet #	Percent Complete					Data In		
Graphite 1	Laboratory		Machining	Mass and Density	Elastic Testing	Mechanical Testing	Thermal Testing	Data Report	Analysis Reports	NDMAS?
PCEA	ORNL	XPC01S8-11	100%	100%	100%	100%	100%	ORNL/TM-2015/765	ORNL/TM-2015/765	NO
PCEA	INL	XPC02S8-7	100%	100%	100%	100%	100%	ECAR-3725	INL/EXT-13-30011	YES
PCEA	INL	XPC01S8-9	100%	100%	100%	100%	100%	ECAR-6111	INL/MIS-23-70949	NO
PCEA	INL	XPC02S8-5	100%	100%	100%	100%	100%	ECAR-6110	INL/MIS-23-70951	NO
PCEA	INL	XPC01D3-35	66%							NO
PCEA	INL	XPC01D3-36	100%	100%	100%	100%	100%	ECAR-3677	INI/EXT-16-39604	YES
PCEA		Multiple Other Billets Available								
NBG-18	INL	635-4	100%	100%	100%	100%	100%	ECAR-3726	INL/EXT-14-33120, INL/EXT- 13-30011	YES
NBG-18	INL	635-14	100%	100%	100%	100%	100%	ECAR-1930	INI/EXT-10-19910, INI/EXT- 13-30011	YES
NBG-18	ORNL	635-6	100%	100%	100%	100%	100%	ORNL/TM-2010/219	ORNL/TM-2010/219	NO
NBG-18		Multiple Other Billets Available								
2114	INL	A20568	100%	100%	100%	100%	100%	ECAR-5798	INL/MIS-22-65680	NO
2114	INL	A20570	100%	100%	100%	100%	100%	ECAR-4322	INL/EXT-14-33120	YES
2114	ORNL	116310	100%	100%	100%	100%	100%	2018/1038, 2019/1256	ORNL/TM-2018/1038, ORNL/TM-2019/1256	YES*
2114		Multiple Other Billets Available								
NBG-17	INL	830-3	100%	100%	100%	100%	100%	ECAR-3727	INI/EXT-14-33120	YES
NBG-17	INL	V104	100%			33%				NO
IG-110	INL	089052-7	100%	100%	100%	100%	100%	ECAR-3521	INL/EXT-14-33120	YES
IG-110	INL	10X69	100%	100%	100%	100%	100%	ECAR-4182	FCAR-4182	NO

### ASME: Component Failure (Martin Metcalfe)

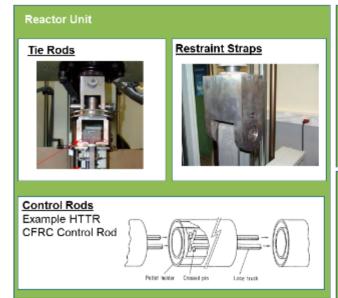
- Composite code development within ASME Section III Division 5 (HHB).
  - Approved in 2019 PBVC version
  - A nonmetallic but wholly different from graphite
    - Highly directional (fiber orientation)
    - Specifically fabricated to design
    - Different failure behavior
  - Initial rules require some modification

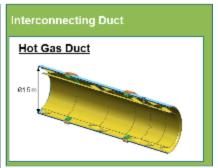






Anticipated applications for composites in HT reactor systems.







### ASME: Design Rules (Andrea Mack)

- Lead Task Group in Nonmetallics Working Group (NWG)
  - Largest commercial vendor participation
- Priority activity in ASME Section III
  - Sec III-5 has designated NWG Design rule changes as a priority activity
  - Striving to get changes into **2025 ed. BPVC**
- Heavy use of Baseline data

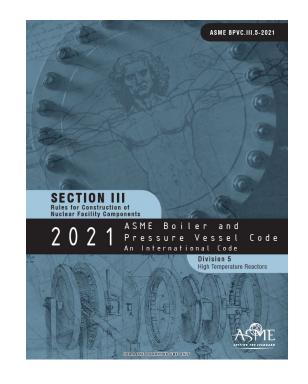
#### **Full Assessment**

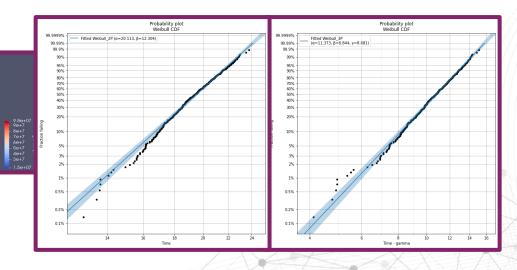
- Disparate flaw distribution
- Tuning  $V_m$  and  $\Delta$
- Mesh refinement
- Location
- Sample size requirements
- Margin

#### **Simplified Assessment**

- Stress terminology
- $R_{tf}$







### ASME: Component Failure (Martin Metcalfe)

- ASME Section III Division 5: Rules for Construction of Nuclear Facility Components (High Temperature Reactors)
  - Damage tolerance and component functionality
  - Review of damage tolerance in the graphite cores of UK power reactors
  - Clarification of terminology associated with component assessment methodologies
- ASME Section XI Division 2: Rules for Inservice Inspection of Nuclear Power Plant Components (Reliability and integrity management)
  - A new supplement covering graphite components in high temperature gas reactors

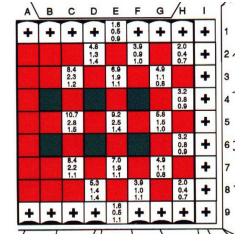


# Vender Irradiation Capsule (Will Windes)

- **ATR**
- "A" Positions
  "B" Positions

  "B" Positions

  "H" Positions
- TD



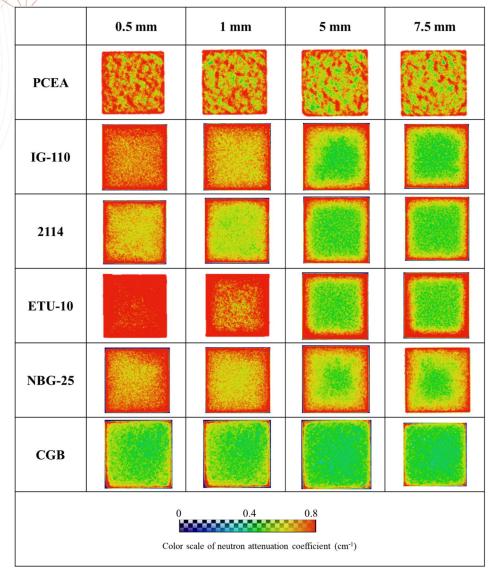
HFR (Petten)

- **HFIR**

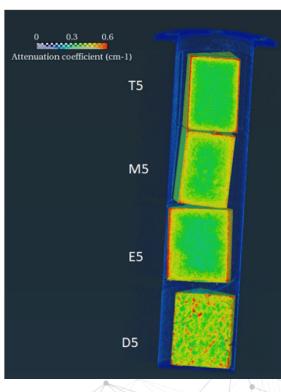
- What is the general idea
- Why is it needed?
- What are our options?
- What do the commercial vendors think?
- What is the next step

### Molten Salt – Graphite Interactions (Nidia Gallego)





- Molten salt is just another degradation mechanism for nuclear graphite
  - Unirradiated and irradiated material behavior still occur
- A status on initial molten-salt interaction studies
  - MS penetration into graphite microstructure
  - Erosion/Abrasion
- New funding molten salt graphite interaction studies (no results yet)
  - Enhance and complete results from these initial studies



### NEUP: New graphite research (Thursday, July 27th, 2023)

#### **Advanced Reactor Component Materials Topics for FY21 NEUP Call**

RC X-Y. Effects of Irradiation Induced Microstructure Change in Graphite

- 1. Multiscale Effects of Irradiation Damage on Nuclear Graphite Properties
  - Aman Haque, Pennsylvania State University
- 2. Quantifying the Dynamic and Static Porosity/Microstructure Characteristics of Irradiated Graphite through Multi-technique Experiments and Mesoscale Modeling
  - Jacob Eapen, North Carolina State University