

#### Modeling and Measurement of Axial Gas Transport in Nuclear Fuels

August 2023

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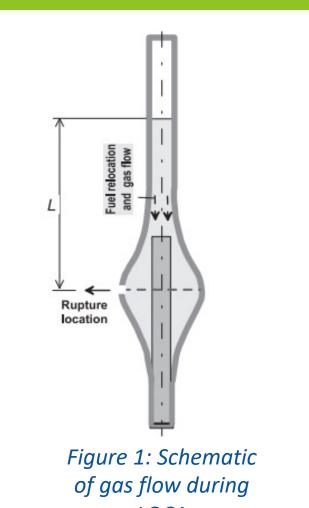
# Modeling and Measurement of Axial Gas Transport in Nuclear Fuels

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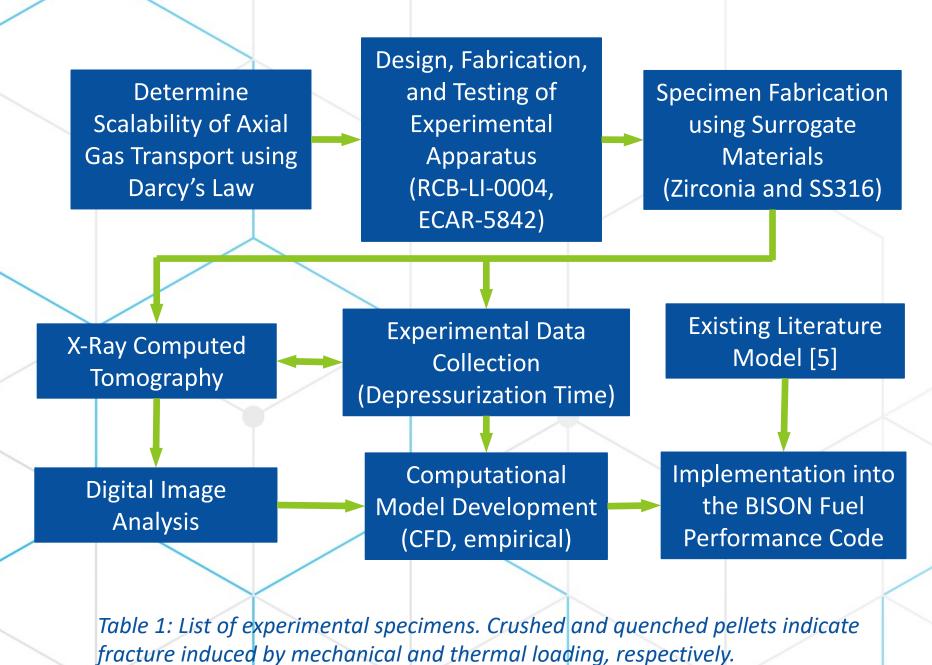
#### Background

- Axial gas transport plays a crucial role in determining pressure equilibrium in nuclear fuel rods
  - Spent fuel storage [1]
  - Loss-of-coolant accident (LOCA) [2]
- Helium-bonded fast reactor fuel rods
- Permeability currently determined a posteriori [3,4]
- Current models assume instantaneous pressure equilibrium and laminar flow



#### Methodology

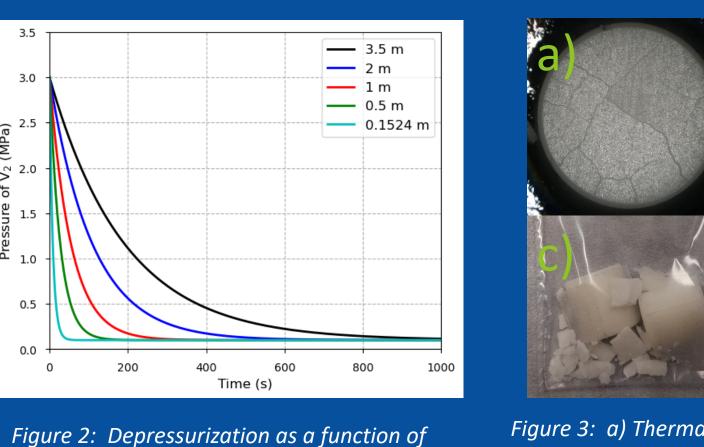
- Verify scalability of axial gas transport to enable full microstructural characterization of specimens for use in model development
- Develop a model for permeability as a function of microstructural features



	Specimen ID	Pellet Status	Number of Pellets	<b>Gap Status</b>	
	Test tube 1 (TT1)	Crushed	4	Open	
	Test tube 2 (TT2)	Crushed	8	Open	
	Control tube 1 (CT1)	Fresh	4	Open	
\	Control tube 2 (CT2)	Quenched	4	Open	
	Control tube 3 (CT3)	Fresh	6	Closed	
	Control tube 4 (CT4)	Quenched	4	Closed	
	Control tube 5 (CT5)	Fresh	8	Closed	
	Control tube 6 (CT6)	Fresh	4	Closed	
	Control tube 7 (CT7)	Fresh	4	Closed	

# Main Research Findings

- Scalability of axial gas transport confirmed
- More accurate fracture patterns obtained by quenching
- Pressure equilibration is not instantaneous
- Gas flow is turbulent for short decay times
- Permeability equation developed as a function of smeared porosity obtained from image analysis



time for varying specimen length

Figure 4: Best-fit curves included and excluding

Forchheimer coefficient for experiment TT1 at an

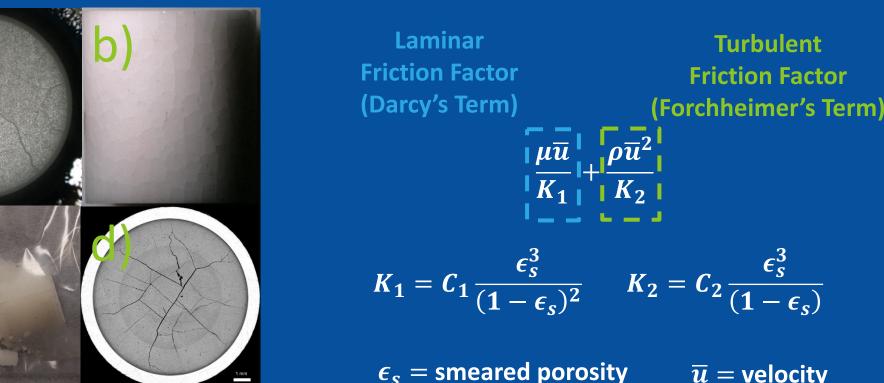
initial pressure of 4.3 MPa

Darcy

Darcy + Forchheimer

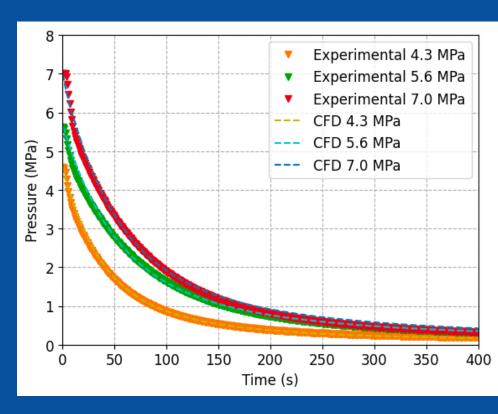
▼ Experimental results

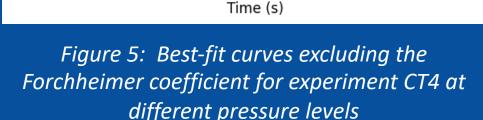
Figure 3: a) Thermal quench (top view) b) thermal quench (side view), c) mechanical crushing, and d) experimental micrograph of irradiated fuel

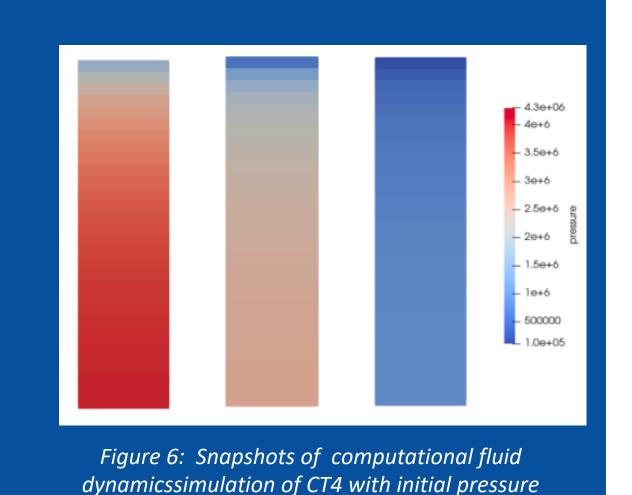


 $\mu = dynamic viscosity$ 

 $C_1$ ,  $C_2$  = pressure-dependent constants







of 4.3 MPa

 $\rho = \mathsf{density}$ 

#### **Publications**

- [1] C. Genoni, et al., "Modeling and Measurement of Axial Gas Transport in Nuclear Fuels," ANS Annual Meeting, June 2023.
- [2] S. Kwon, et al., "Fabrication of Surrogate Oxide Spent Fuel with Various Cracking Patterns and the Design of an Axial Gas Transport Apparatus," to be submitted to the Journal of Nuclear Materials.
- [3] C. Genoni, et al., "Investigation of the Impact of Non-Uniform Permeability on Axial Gas Transport within Light Water Reactor Fuel Rods during a Loss-Of-Coolant Accident," to be submitted to Nuclear Engineering and Design.
- [4] T. Bergomi, "Fuel Pellets Three-Dimensional Properties Reconstruction Exploiting Image Analysis: A Bridge Between Experiments and Modeling", Masters Thesis, Politecnico Di Milano, expected 2024.

## **Specimen Fabrication and Experimental Apparatus**

- Decay of inlet pressure measured as a function of
  - 4.3, 5.6, and 7 MPa analyzed
- Quenching and mechanical crushing used to induce fracture

Mylar wrapping used to preserve pellet position

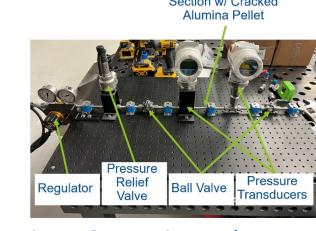




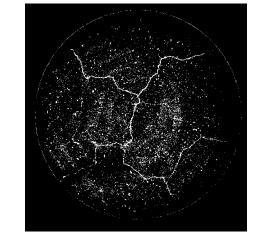


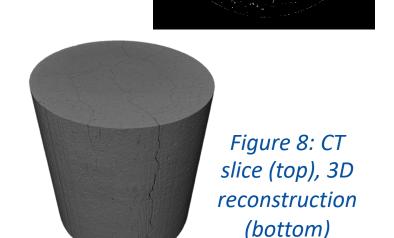
Figure 7: Photo of (a) pellets with mylar wrapping, (b) mechanical compression of individual pellet, and (c) resulting separation-crack formation

### **Digital Image Analysis**

- **Hundreds** of two-dimensional (2D) images obtained per specimen by x-ray computed tomography (CT)
- Image analysis used to estimate features

  - **Crack tortuosity**
  - Specific surface
- Porosity distribution
- **3D reconstruction** of the pellets can be performed





#### References

- [1] V. Rondinella et al., TopFuel 2015. JRC94524.
- [2] W. Wiesenack, et al. International Conference on the Physics of Reactors, 2008.
- [3] R. Montgomery and R. N. Morris, doi: doi.org/10.1016/j.jnucmat.2019.05.041. [4] S. J. Dagbjartsson, et al., TREE-NUREG-1158, 1977.
- [5] Khvostov, G., et al., doi: doi.org/10.1016/j.nucengdes.2011.03.003.

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